

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

W. L. STEWART
VICE PRESIDENT
NUCLEAR OPERATIONS

July 3, 1986

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Lester S. Rubenstein, Director
PWR Project Directorate No. 2
Division of PWR Licensing-A
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Serial No. 86-426
NO/TAH/vlh
Docket Nos. 50-338
50-339
License Nos. NPF-4
NPF-7

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION UNIT NOS. 1 AND 2
LICENSE AMENDMENT FOR CORE UPDATE

By letter dated May 2, 1985, Virginia Electric and Power Company requested an amendment to the North Anna Unit 1 and 2 Technical Specifications to allow operation with a nominal core thermal power rating of 2893 Mwt. By letter dated October 23, 1985, the NRC Staff requested additional information concerning the proposed core uprating. Our letter, dated February 6, 1986, provided the response to the request.

This response was based in part on the assumption that the North Anna Emergency Operating Procedures would be revised based on Revision 1 to the Westinghouse Owners Group Emergency Response Guidelines (Rev. 1 WOG/ERGs) prior to the anticipated date of implementation of the core uprating. However, as a result of a recent audit by the NRC, of the North Anna Emergency Operating Procedure generation process, the implementation of the Rev. 1 WOG/ERG based procedures has been delayed from June 30, 1986 until April 30, 1987. The purpose of this letter is to inform you that we have performed a review to insure the accuracy of our responses to the October 23, 1985 request for additional information and to provide you with the results of the review.

The review was performed to identify any assumptions in our response which are inconsistent with operator actions directed by the current versus the revised Emergency Operating Procedures. Also, proposed corrections were to be provided if any discrepancies were identified. The responses to questions 1.b and 2.c of the staff's request were determined to be dependent upon operator actions and therefore are potentially affected.

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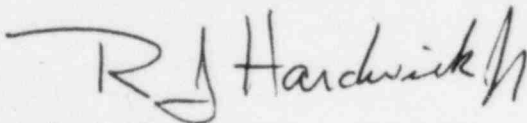
VIRGINIA ELECTRIC AND POWER COMPANY TO Mr. Harold R. Denton

The response to item 1.b concerned a locked rotor accident with a stuck open steam generator PORV. To assure consistency with our response, it will be necessary to modify the existing procedures by changing step 7 of EP-2 to verify isolation of (or manually close) the SG PORVs and the SG Blowdown in addition to isolating all feedwater, as the current procedure requires.

Item 2.c concerned small break LOCAs from 1000 psig without accumulators. Our response to this item remains viable with the existing procedures. The Critical Safety Function Status Trees for Core Cooling in both Rev. 1 WOG/ERG based procedures and the existing procedures, monitor the Core Exit Thermocouples (CETs) and the Reactor Vessel Liquid Inventory System (RVLIS). If the CETs exceed 700°F or if RVLIS indicates core uncover, the procedures transitioned to, in both the current and the upgraded versions will direct the operator to depressurize the Reactor Coolant System via steam generator secondary side depressurization. This is consistent with the February 6, 1986 response.

The conclusion from this review is that even though the responses to the NRC questions were written assuming that Rev. 1 WOG/ERG based EOPs would be implemented, they remain applicable for the current procedures during the interim period. However, the change described above will be made to the existing procedures prior to implementation of the core uprating in order to insure consistency with our response of February 6, 1986.

Very truly yours,



+ W. L. Stewart

VIRGINIA ELECTRIC AND POWER COMPANY TO Mr. Harold R. Denton

cc: Dr. J. Nelson Grace
Regional Administrator
NRC Region II

Mr. J. L. Caldwell
NRC Senior Resident Inspector
North Anna Power Station

Mr. Leon B. Engle
NRC North Anna Project Manager
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