

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 2, 1998

Mr. W. R. McCollum, Jr.
Vice President
Duke Power Company
Oconee Nuclear Site
P. O. Box 1439
Seneca, South Carolina 27679

SUBJECT:

ACCEPTANCE OF DUKE POWER COMPANY'S RESPONSE TO NUCLEAR REGULATORY COMMISSION BULLETIN 96-04

Dear Mr. McCollum:

On August 19, 1996, as supplemented November 3 and December 31, 1997, Duke Power Company responded to Nuclear Regulatory Commission Bulletin 96-04 for the NUHOM 24P spent fuel storage system used at the Oconee Nuclear Site. This is to inform you that NRC reviewed your response and found it acceptable.

In your response, you incorporated and concurred with the bulletin response from VECTRA Technologies, Inc. (now Transnuclear West, Inc.). On December 5, 1997, NRC informed you that it had accepted VECTRA's response and issued a safety evaluation (dated November 17, 1997). As discussed in the safety evaluation, hydrogen could be generated in the NUHOMS-24P due to the corrosion of the flame-sprayed aluminum coating in borated water. Consequently, VECTRA recommended setting a safe upper limit of 2.4% hydrogen by volume for the hydrogen concentration in the dry shielded canister (DSC) during welding or cutting operations. The staff accepted this recommendation and further concluded that DSCs with flame-sprayed aluminum should be monitored continuously for hydrogen during welding or cutting.

Accordingly, in your December 31, 1997, submittal, you agreed to revise the cask loading and unloading procedures to include continuous monitoring of the DSC vapor space during welding, grinding, or cutting of DSCs with flame-sprayed aluminum. In your November 3, 1997, submittal, you also agreed to revise the procedures to include VECTRA's recommended safety limit of 2.4% hydrogen by volume. NRC's understanding is that you will complete these procedure changes before loading or unloading any DSCs in the future. Please note that the cask loading and unloading procedures at Oconee may be the subject of a future NRC inspection.

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If you have any questions regarding this matter, you may contact me at (301) 415-8531 or Timothy McGinty at (301) 415-8580.

Sincerely,

[original signed by /s/]

Marissa G. Bailey, Project Manager Spent Fuel Licensing Section Spent Fuel Project Office Office of Nuclear Material Safety and Safeguards

Dockets 72-4, 50-269/270/287

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Oconee Nuclear Station Units 1, 2, and 3

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