NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Callaway Plant Unit 1	0 5 0 0 0 4 8 3	1 OF 0 2

TITLE (4)
Low Pressurizer Pressure Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 6	0 6	8 6	8 6	0 1 9	0 0	0 7	0 3	8 6		0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
W. R. Robinson - Supterintendent, I&C	AREA CODE 3 1 4 — 6 7 6 - 8 2 9 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 6/6/86 at 1817 CDT a reactor trip occurred on low pressurizer pressure. The plant was in Mode 1 at 100% power at the time of the event.

Technicians had calibrated pressurizer pressure transmitter BB-PT-458, which shares a common sensing line with pressure transmitter BB-PT-457. While restoring BB-PT-458 to service, a deviation from the procedure resulted in both transmitters spiking low when leakage occurred through a test isolation valve.

To prevent recurrence, technicians were given definitive guidelines as to times when procedures are to be "in hand" vice reliance on knowledge considered within their normal job scope. The importance of verbatim compliance with procedures was emphasized, and the procedure for returning the transmitter to service will be revised to check for isolation valve leakage. Additionally, progressive discipline was administered to the appropriate personnel.

All plant safety equipment performed as designed. At no time did this event pose a threat to the public health or safety.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Callaway Plant Unit 1	0 5 0 0 0 4 8 3	8 6	— 0 1 9	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 6/6/86 at 1817 CDT a reactor trip occurred on low pressurizer pressure. The plant was in Mode 1 at 100% power at the time of the event. Operators performed the appropriate emergency operating procedures and the plant reached a stable condition at 1835 on 6/6/86.

I&C technicians had completed replacing and calibrating Reactor Coolant System (RCS) Pressurizer Pressure Transmitter BB-PT-458,(1) which shares a common sensing line with RCS Pressurizer Pressure Transmitter BB-PT-457.(1) While returning the transmitter to service, the technician pressurized BB-PT-458 to system pressure, closed the test isolation valve and opened the sensing line isolation valve. During removal of test equipment from BB-PT-458, a pressure surge occurred due to leakage through the test isolation valve, causing BB-PT-458 to spike low. The procedure for returning the transmitter to service requires that the test equipment used to pressurize the transmitter to system pressure be removed prior to opening the sensing line isolation valve. Had the sensing line isolation valve been closed, only BB-PT-458 would have been affected by this leakage. However, due to the common sensing line and the open sensing line isolation valve, BB-PT-457 also spiked low, thus satisfying the 2 out of 4 low pressurizer pressure reactor trip logic.

To prevent recurrence, technicians were given definitive guidelines as to times when procedures are to be "in hand" vice reliance on knowledge considered within their normal job scope. The importance of verbatim compliance with procedures was emphasized, and the procedure for returning the transmitter to service will be revised to include a check of transmitter remote indication to detect leakage in the isolation valve. Additionally, progressive discipline was administered to the appropriate personnel.

All plant safety equipment performed as designed. At no time did this event pose a threat to the public health or safety.

Previous occurrences: LER 84-035-00 transmitted via ULNRC-939 dated 10/6/84 and LER 84-040-00 transmitted via ULNRC-950 dated 10/15/84 reported similar events involving common tap instrumentation in the Feedwater System. LER 85-016-00 transmitted via ULNRC-1073 dated 4/2/85 reported an event involving BB-PT-457 and BB-PT-458; however, the event was the result of a sensing line valve misfunction and not a personnel error.

Footnote

(1) IEEE Standard 803-A-1983 Component - PT
IEEE Standard 805-1983 System - AB

UNION ELECTRIC

Callaway Plant

July 3, 1986

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-1337

Gentlemen:

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 86-019-00
LOW PRESSURIZER PRESSURE REACTOR TRIP

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning a low pressurizer pressure reactor trip initiated during restoration of a pressurizer pressure transmitter to service.

GL Randolph

G. L. Randolph
Manager, Callaway Plant

Dunn JED JMK
WRR/JED/JMK/drs
Enclosure

cc: Distribution attached

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1/1

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