

November 24, 1997

Mr. Donald A. Reid
Senior Vice President, Operations
Vermont Yankee Nuclear Power Corporation
185 Old Ferry Road
Brattleboro, VT 05301

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - GENERIC LETTER 95-07,
"PRESSURE-LOCKING AND THERMAL-BINDING OF SAFETY-RELATED POWER-
OPERATED GATE VALVES" - VERMONT YANKEE NUCLEAR POWER STATION
(TAC NO. M93533)

Dear Mr. Reid:

On August 17, 1995, the NRC issued Generic Letter (GL) 95-07, "Pressure-Locking and Thermal-Binding of Safety-Related Power-Operated Gate Valves," to request that licensees take actions to ensure that safety-related power-operated gate valves that are susceptible to pressure-locking or thermal-binding are capable of performing their safety functions.

The NRC staff has reviewed your GL 95-07 submittals dated February 8, and June 21 and 26, 1996, for the Vermont Yankee Nuclear Power Station. Based on its review, the staff finds that responses to the enclosed request for additional information are needed before we can complete our review.

Please provide your responses within 30 days from the date of this letter. If you have any questions regarding this matter, please contact me at (301) 415-1496.

Sincerely,

Original signed by

Kahtan N. Jabbour, Sr. Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-271

Enclosure: Request for Additional
Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION
VERMONT YANKEE NUCLEAR POWER STATION
RESPONSES TO GENERIC LETTER 95-07,
"PRESSURE-LOCKING AND THERMAL-BINDING
OF SAFETY-RELATED POWER-OPERATED GATE VALVES"

Docket No. 50-271

1. The February 8, 1996 submittal states that valves V10-15A/B/C/D, V10-17, and V10-18, Residual Heat Removal Pump Suction, have a safety function to close and do not have a safety function to open. Therefore, Vermont Yankee Nuclear Power Corporation did not include these valves in the scope of GL 95-07. These valves are opened to initiate shutdown cooling in order to cooldown the unit following a reactor shutdown. Does the licensing basis for Vermont Yankee Nuclear Power Station require that the unit be cooled down following a reactor shutdown? If so, what safety-related systems are used to accomplish the cooldown?

Valves that are required to open in order to meet 10 CFR 50, Appendix R, requirements are not in the scope of GL 95-07. The NRC staff is not requesting that you address Appendix R shutdown/cooldown requirements when responding to this request for additional information.

2. The February 8, 1996 submittal states that valves V10-26A, Residual Heat Removal Drywell Spray, V23-14, High Pressure Coolant Injection Steam Admission, and V23-20, High Pressure Coolant Injection Test, would be modified to prevent pressure locking during the 1996 refueling outage. Please confirm that these modifications were completed as scheduled.
3. The February 8, submittal states that valves V10-31A/B, Residual Heat Removal Containment Spray, are not susceptible to pressure locking. Discuss why these valves are not susceptible to pressure locking. If your pressure locking evaluation assumes that upstream valves V10-26A/B do not leak by the seat, describe seat leakage test results that demonstrate that the valves do not leak.

Enclosure