

January 20, 1999

Mr. James W. Langenbach, Vice President
and Director - TMI
GPU Nuclear, Inc.
P.O. Box 480
Middletown, PA 17057

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING GPU NUCLEAR,
INC., TECHNICAL SPECIFICATION CHANGE REQUEST (TSCR) NO. 277,
ONCE THROUGH STEAM GENERATOR (OTSG) CYCLE 13 (13R) INSERVICE
INSPECTIONS FOR THE THREE MILE ISLAND NUCLEAR STATION, UNIT 1
(TMI-1) (TAC NO. MA4234)

Dear Mr. Langenbach:

The staff has reviewed your November 25, 1998, license amendment application regarding TSCR No. 277, for OTSG inservice inspection during 13R for TMI-1. The staff has determined that it will need the additional information identified in the enclosure to complete its review. As discussed with and agreed to by your staff, please respond to this request for additional information within 3 weeks of receipt in order for the staff to remain on schedule with its review.

Sincerely,

/s/

Timothy G. Colburn, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Docket No. 50-289

Enclosure: Request for Additional Information

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, reading "Timothy G. Colburn".

Timothy G. Colburn, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-289

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cc w/encl: See next page

J. Langenbach
Three Mile Island Nuclear Station, Unit No. 1

cc:

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**REQUEST FOR ADDITIONAL INFORMATION
THREE MILE ISLAND, UNIT 1
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 277
CYCLE 13 STEAM GENERATOR ID IGA INSPECTION ACCEPTANCE CRITERIA**

By letter dated November 25, 1998, GPU Nuclear, Inc., submitted a request to change the Technical Specification surveillance requirements for steam generator inservice inspections for Three Mile Island, Unit 1 (TMI-1) cycle 13 refueling (13R) outage examinations which would be applicable for one cycle of operation only. Specifically, the change requested is associated with inspection acceptance criteria for inside diameter intergranular attack (ID IGA) of the steam generator tubes. The staff has reviewed the request and determined additional information is necessary to complete the review. The required information is listed below.

1. The November 25, 1998, letter stated the metallurgical and chemical analyses performed on the 12R outage pulled tubes suggest strongly that there was no progression in ID volumetric degradation. The conclusion that ID volumetric degradation is non-active degradation is a significant aspect of the technical basis supporting this amendment request. Therefore, please submit a summary of the metallurgical and chemical analyses performed, and the results of the analyses which enabled you to determine that the ID volumetric degradation is not progressing.
2. Provide a discussion on the inspections planned (rotating pancake coil and bobbin probe) for the 13R outage as compared to previous outage inspections (specifically, the 11R and 12R outages). This information is needed to assess the credibility of flow growth rate data as determined from a comparative analysis of the 12R and 11R inspection results. The discussion should focus on the equipment not the inspection scopes, and should be geared towards similarities and differences. Include the following items in addition to other details you determine to be relevant: describe the probes used (for detection, sizing and comparisons to previous outage data); describe how probe wear is evaluated and controlled; describe how calibrations are controlled (e.g., calibration standards); and describe evaluations of data analyst uncertainties if performed.
3. A one-cycle technical specification change was previously requested and granted for ID IGA degradation for the 12R cycle and outage. Please discuss your reasons, both technical and regulatory, for requesting another one-cycle change for 13R.