

Docket Number 50-346

License Number NPF-3

Serial Number 2499

November 6, 1997

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555-0001

Subject: Initial Response to NRC Generic Letter 97-04: Assurance of
Sufficient Net Positive Suction Head for Emergency Core
Cooling and Containment Heat Removal Pumps

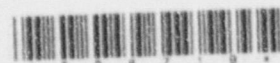
Ladies and Gentlemen:

On October 7, 1997, the Nuclear Regulatory Commission (NRC) issued
Generic Letter (GL) 97-04, requesting that nuclear power plant licensees
review their current design-basis analyses used to determine the available
net positive suction head (NPSH) for the emergency core cooling (including
core spray and decay heat removal) and containment heat removal pumps
that meet either of the following criteria:

- (1) pumps that take suction from the containment sump or suppression pool
following a design-basis Loss of Coolant Accident (LOCA) or secondary
line break, or
- (2) pumps used in "piggyback" operation that are necessary for recirculation
cooling of the reactor core and containment (that is, pumps that are
supplied by pumps which take suction directly from the sump or
suppression pool).

Based on this review, licensees are requested to provide the information
outlined below to the NRC by January 5, 1998:

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1. Specify the general methodology used to calculate the head loss associated with the ECCS suction strainers.
2. Identify the required NPSH and the available NPSH.
3. Specify whether the current design-basis NPSH analysis differs from the most recent analysis reviewed and approved by the NRC for which a safety evaluation was issued.
4. Specify whether containment overpressure (i.e., containment pressure above the vapor pressure of the sump or suppression pool fluid) was credited in the calculation of available NPSH. Specify the amount of overpressure needed and the minimum overpressure available.
5. When containment overpressure is credited in the calculation of available NPSH, confirm that an appropriate containment pressure analysis was done to establish the minimum containment pressure.

Generic Letter 97-04 required that licensees submit an initial response to the NRC by November 6, 1997, indicating whether or not the above requested information will be submitted within the requested time period. Toledo Edison plans to submit the requested information regarding the Davis-Besse Nuclear Power Station, Unit Number 1 to the NRC by January 5, 1998. Should you have any questions or require additional information, please contact Mr. James L. Freels, Manager-Regulatory Affairs, at (419) 321-8466.

Very truly yours,

FWK/laj

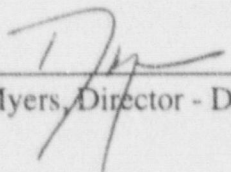
cc: A. B. Beach, Regional Administrator, NRC Region III
S. J. Campbell, DB-1 NRC Senior Resident Inspector
A. G. Hansen, DB-1 NRC/NRR Project Manager
Utility Radiological Safety Board

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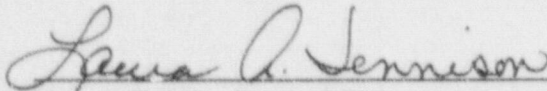
RESPONSE
TO
NRC GENERIC LETTER 97-04
FOR THE
DAVIS-BESSE NUCLEAR POWER STATION
UNIT NUMBER 1

This letter is submitted pursuant to 10 CFR 50.54(f) and contains information pursuant to NRC Generic Letter 97-04, "Assurance of Sufficient Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal Pumps" for the Davis-Besse Nuclear Power Station, Unit Number 1.

For: John K. Wood, Vice President - Nuclear

By: 
T. J. Myers, Director - DB Nuclear Support Services

Sworn to and subscribed before me this 6th day of November, 1997


Notary Public, State of Ohio

LAURA A. JENNISON
Notary Public, State of Ohio
My Commission Expires 8-15-2001