



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

OREGON STATE UNIVERSITY

DOCKET NO. 50-243

AMENDMENT TO AMENDED FACILITY OPERATING LICENSE

Amendment No. 16
License No. R-106

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that
 - A. The application for an amendment to Amended Facility Operating License No. R-106 filed by Oregon State University (the licensee) on November 25, 1997, as supplemented on December 19, 1997, conforms to the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the regulations of the Commission as stated in Chapter I of Title 10 of the *Code of Federal Regulations* (10 CFR);
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance that (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) such activities will be conducted in compliance with the regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. This amendment is issued in accordance with the regulations of the Commission as stated in 10 CFR Part 51, and all applicable requirements have been satisfied; and
 - F. Prior notice of this amendment was not required by 10 CFR 2.105, and publication of notice for this amendment is not required by 10 CFR 2.106.

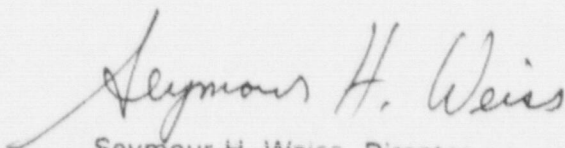
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Amended Facility Operating License No. R-106 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 16, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Seymour H. Weiss, Director
Non-Power Reactors and Decommissioning
Project Directorate
Division of Reactor Program Management
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A, Technical
Specifications Changes

Date of Issuance: January 28, 1998

ENCLOSURE TO LICENSE AMENDMENT NO. 16

AMENDED FACILITY OPERATING LICENSE NO. R-106

DOCKET NO. 50-243

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

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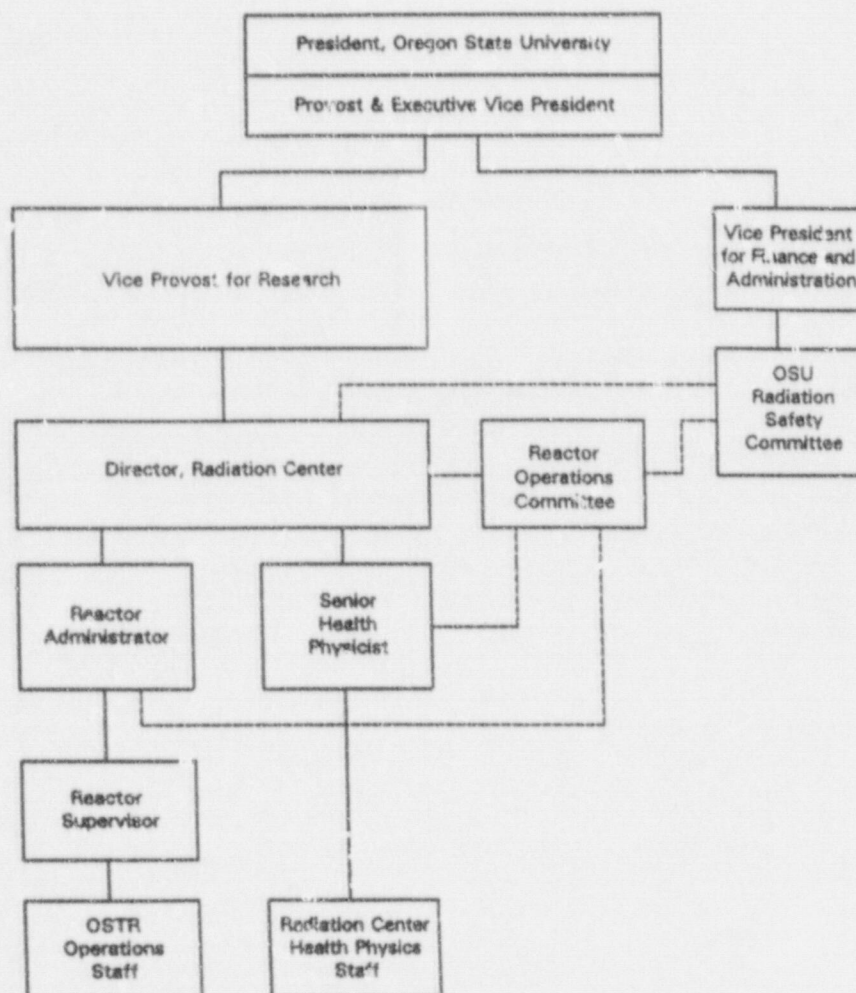
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6. ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION

- a. The facility shall be under the direct control of the Radiation Center Director or a licensed senior operator designated by the Director to be in direct control. The Radiation Center Director shall be responsible to Oregon State University's Vice Provost for Research for the safe operation and maintenance of the reactor and its associated equipment. The Radiation Center Director, or an individual appointed by the Director, shall be responsible for assuring that all operations are conducted in a safe manner and within the limits prescribed by the facility license and the requirements of the Reactor Operations Committee. The Radiation Center Director shall enforce rules for the protection of personnel against radiation.
- b. The safe operation of the OSTR shall be related to the University Administration as shown in the following chart:



_____ Normal administrative reporting channel

_____ Technical review (as applicable), communications and assistance

- g. Offsite environmental monitoring surveys.
- h. Fuel inventories and transfers.
- i. Facility radiation and contamination surveys.
- j. Radiation exposures for all personnel.
- k. Updated, corrected, and as-built drawings of the facility.

6.7 REPORTING REQUIREMENTS

In addition to the requirements of applicable regulations, and in no way substituting therefore, reports shall be made to the NRC as follows:

- a. A report within 24 hours by telephone or fax to the NRC Operations Center of:
 - 1. Any accidental release of radioactivity above applicable limits in unrestricted areas, whether or not the release resulted in property damage, personal injury, or exposure;
 - 2. Any violation of a safety limit;
 - 3. Operation with a safety system setting less conservative than specified in Section 2.2, Limiting Safety System Settings;
 - 4. Operation in violation of a Limiting Condition for Operation;
 - 5. Failure of a required reactor or experiment safety system component which could render the system incapable of performing its intended safety function unless the failure is discovered during maintenance tests or periods of reactor shutdown;
 - 6. Any unanticipated or uncontrolled change in reactivity greater than \$1.00;
 - 7. An observed inadequacy in the implementation of either administrative or procedural controls, such that the inadequacy could have caused the existence or development of a condition which could result in operation of the reactor outside the specified safety limits; and
 - 8. A measurable release of fission products from a fuel element.
- b. A report within 14 days in writing to the NRC, Document Control Desk, Washington, D.C.
 - 1. Any accidental release of radioactivity above permissible limits in unrestricted areas, whether or not the release resulted in property damage, personal injury, or exposure; the written report (and, to the extent possible, the preliminary telephone or fax report) shall describe, analyze, and evaluate safety implications, and outline the corrective measures taken or planned to prevent reoccurrence of the event;

2. Those events reported as required by Sections 6.7.a.2 through 6.7.a.8.
- c. A report within 30 days in writing to the NRC, Document Control Desk, Washington, D.C.
1. Any significant variation of measured values from a corresponding predicted or previously measured value of safety-connected operating characteristics occurring during operation of the reactor;
 2. Any significant change in the transient or accident analyses as described in the Safety Analysis Report;
 3. Any changes in facility organization or personnel; and
 4. Any observed inadequacies in the implementation of administrative or procedural controls such that the inadequacy causes or could have caused the existence or development of an unsafe condition with regard to reactor operations.
- d. A report within 90 days after completion of starting testing of the reactor (in writing to the NRC, Document Control Desk, Washington, D.C.) upon receipt of a new facility license, or an amendment to the license authorizing an increase in reactor power level, describing the measured values of the operating conditions or characteristics of the reactor under the new conditions including:
1. An evaluation of facility performance to date in comparison with design predictions and specifications.
 2. A reassessment of the safety analysis submitted with the license application in light of measured operating characteristics when such measurements indicate that there may be substantial variance from prior analysis.
- e. An annual report by November 1 of each year (in writing to the NRC, Document Control Desk, Washington, D.C.).
1. A brief summary of operating experience including experiments performed and changes in facility design, performance characteristics and operating procedures related to reactor safety occurring during the reporting period, and results of surveillance test and inspections.
 2. A tabulation showing the energy generated by the reactor (in megawatt-hours), hours reactor was critical, and the cumulative total energy output since initial criticality.
 3. The number of emergency shutdowns and inadvertent scrams, including reasons therefore.
 4. Discussion of the major maintenance operations performed during the period, including the effect, if any, on the safety of the operation of the reactor and the reasons for any corrective maintenance required.