

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) LaSalle County Station Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 7 3				PAGE 15 1 OF 0 3					
TITLE (4) Reactor Coolant Pressure Boundary Pipe Leaks on LPCI Lines																			
EVENT DATE (5)				LER NUMBER (6)				REPORT DATE (7)				OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENT NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)						
0 5	2 1	8 6	8 6	0 1	9	0 0	0 6	1 6	8 6					0 5 0 0 0					
OPERATING MODE (9) 4				THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)															
POWER LEVEL (10) 0 1 0 0				20.402(b)				20.405(e)				80.73(a)(2)(iv)				73.71(b)			
				20.405(a)(1)(i)				80.38(a)(1)				80.73(a)(2)(v)				73.71(a)			
				20.405(a)(1)(ii)				80.38(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 308A)			
				20.405(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)							
				20.405(a)(1)(iv)				X 80.73(a)(2)(ii)				80.73(a)(2)(viii)(B)							
				20.405(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(ix)							
LICENSEE CONTACT FOR THIS LER (12)																			
NAME D. Zoloty, Technical Staff Engineer, extension 421										TELEPHONE NUMBER 8 1 5 3 5 7 - 6 7 6 1									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS									
X	B	P	S	X	Y														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO							

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Two through wall leaks were identified during the ASME Section XI leakage test on the Class 1 portion of the Residual Heat Removal (RHR) "B" and "C" Low Pressure Coolant Injection (LPCI) loops. The leakage was coming from preservice plug-type repair welds in the vicinity of the 1E12-F092B and 1E12-F092C valves. The leaking plugs will be repaired during the present Unit 1 refuel outage. The unit was in Cold Shutdown.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
LaSalle County Station Unit 1	05000373	86	019	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

I. EVENT DESCRIPTION:

On May 21 and 22, 1986, an ASME Section XI leakage test was conducted on the LaSalle Unit 1 primary system piping while the unit was in Cold Shutdown. With a minimum test pressure of 1020 psig in the reactor vessel steam dome, leakage was identified coming from the Residual Heat Removal (RHR, BO) "B" and "C" loop injection piping in the vicinity of the 1E12-F092B and C valves. Upon removal of the pipe insulation, slight leakage was found to be seeping through the pipe wall in the area of a preservice repair weld. This weld is located on the section of piping between the F092 valves and the reactor vessel. Investigation has shown the repair to be a 3/4" threaded plug inserted flush with the pipe ID with approximately 2 tenths inch of weld metal layed over the plug outer surface to increase the repair thickness to slightly greater than the 0.844" pipe wall thickness. Subsequent to finding leaks on the "B" and "C" loops, the insulation was removed to expose the same repair weld on the "A" loop of RHR. This plug repair was not found to be leaking and appeared to be in a sound condition. The pipe and 3/4" plug on all 3 loops are fabricated from carbon steel material.

II. CAUSE:

The leakage is postulated to be due to entrapped slag or some other defect in the plug seal weld.

III. PROBABLE CONSEQUENCES OF OCCURRENCE:

The as found condition of the leaking plug repairs has had no significant effect on the system status. Postulating the situation where a 3/4" plug experienced complete failure, the resulting leakage would easily fall within the capability of the ECCS systems. Any leakage from these plugs would be readily detected by the drywell floor sump leakage monitors before complete failure occurred.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104
EXPIRES 9/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (3)			PAGE (3)			
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LaSalle County Station Unit 1	0500037386	-	019	-	00	03	OF	03

TEXT (If more space is required, use additional NRC Form 288A's) (17)

IV. CORRECTIVE ACTIONS:

During the present refueling outage, the existing two flawed seal welds will both be enclosed using a 1" nominal diameter 6000 # half coupling attached to the outside of the RHR piping with a full penetration weld. Following a visual inspection of the weld root for penetration, the coupling will be capped. At the unit's second refueling outage, the system will be drained, the cap removed from the half coupling, and the original threaded plug removed from the pipe wall. The half coupling cap will then be replaced. Also during the Unit 1 second refueling outage, the non-leaking plug on the "A" RHR LPCI line will be repaired (AIR 373-200-86-06100). The three similar plugs on the Unit 2 LPCI lines will be repaired during the Unit 2 first refueling outage (AIR 373-200-86-06101).

V. PREVIOUS OCCURRENCES:

None.

VI. NAME AND TELEPHONE NUMBER OF PREPARER:

D. Zoloty, Technical Staff Engineer, 815/357-6761, extension 421.



Commonwealth Edison
LaSalle County Nuclear Station
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Telephone 815/357-6761

June 16, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #86-019-00, Docket #050-373 is being submitted to your office in accordance with 10CFR 50.73.

R. D. Bieders
for G. J. Diederich
Station Manager
LaSalle County Station

GJD/DRR/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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