

**Florida  
Power**  
CORPORATION

June 18, 1986  
3F0686-12

Mr. H. R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

SUBJECT: Crystal River Unit 3  
Docket No. 50-302  
Operating License No. DPR-72  
Technical Specification Change Request No. 145  
High Pressure Trip Setpoint and Anticipatory Reactor Trip

Dear Sir:

Florida Power Corporation (FPC) hereby submits the enclosed three (3) originals and forty (40) copies of Technical Specification (TS) Change Request No. 145 requesting amendment to Appendix A of Operating License No. DPR-72. As part of this request the proposed replacement pages for Appendix A are enclosed.

Attachment A requests an increase in the current TS Setpoint for reactor high pressure trip from 2300 PSIG to 2355 PSIG. Attachment B updates and replaces NUREG-0737 Technical Specifications regarding anticipatory reactor trips previously submitted as part of Technical Specification Change Request Number 82. The new operating limits established by this change have been recommended by the Babcock & Wilcox Owner's Group Trip Reduction and Transient Response Improvement Program (nicknamed STOP TRIP). Your prompt attention to consideration of these changes would be an important step in supporting the efforts of the B&W STOP TRIP Program.

There are currently no CR-3 TS's on anticipatory reactor trip (ART). FPC personnel actively involved in the industry/staff technical specification improvement program (TSIP) recognize that ART specifications fall into a category where it is likely that such specifications will be moved to another document under 10CFR50.59 control. Anticipating that ART specifications will not remain in TS's after TSIP implementation, Attachment B is provided separately from Attachment A to facilitate staff action. Action on this request can be utilized as part of the lead plant concept being encouraged by NRC/TSCB.

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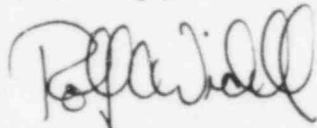
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Both ART functions are currently installed and functioning at CR-3. Initially both were automatically armed at 20% full reactor power and above. Based on the results of Babcock and Wilcox topical reports and their acceptance by the NRC staff in Safety Evaluations issued in April 1986 (see Attachments A and B for references), FPC has raised the arming threshold for ART on turbine trip to 30% full reactor power. When the technical specification for reactor high pressure trip is raised to 2355 PSIG, it is anticipated that FPC will raise the ART turbine trip arming threshold to 45% full reactor power. The arming threshold for ART on trip of both main feedwater pumps remains at or below 20% full reactor power.

An amendment application fee, check number 840487 of one hundred fifty dollars (\$150.00), as required by 10 CFR 170, has been included with this change request.

Sincerely,



Rolf C. Widell  
Manager, Nuclear Operations  
Licensing and Fuel Management

SCP:jk

Enclosures

cc: Dr. J. Nelson Grace  
Regional Administrator, Region II  
U. S. Nuclear Regulatory Commission  
101 Marietta St., N.W., Suite 2900  
Atlanta, GA 30323