

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 1 1										PAGE (3) 1 OF 013	
TITLE (4) LEAKING VALVES CAUSE DRYWELL UNIDENTIFIED LEAKAGE TO EXCEED TECH. SPECS. LIMIT																					
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)						
11	26	85	85	041	0	10	61	18							0 5 0 0 0						
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																					
OPERATING MODE (9)		1		20.402(b)		20.406(c)		80.73(a)(2)(iv)		73.71(b)											
POWER LEVEL (10)		0.1917		20.406(a)(1)(i)		80.36(c)(1)		80.73(a)(2)(v)		73.71(c)											
				20.406(a)(1)(ii)		80.36(c)(2)		80.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
				20.406(a)(1)(iii)		80.73(a)(2)(ii)		80.73(a)(2)(vii)(A)													
				20.406(a)(1)(iv)		80.73(a)(2)(iii)		80.73(a)(2)(vii)(B)													
				20.406(a)(1)(v)		80.73(a)(2)(iii)		80.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																					
NAME Raymond D. Baker, Nuclear Licensing Manager - Hatch								TELEPHONE NUMBER 410 14 512 16 17 10 116													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NPDs												
X	AIDV	111	C161814	Y																	
X	S1JV	111	R131414	Y																	
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 11/26/85 at approximately 1500 CST, with reactor power at 2367 MWt (approximately 97% rated power), plant personnel noted that the unidentified leakage into the Drywell floor drain had exceeded the limit of 5 gpm (gallons per minute) when averaged over the last 24 hours as specified by Technical Specifications Section 3.6.G.1.a. The required 4 hour LCO (limiting condition for operation) was initiated as required by Technical Specifications Section 3.6.G.3.a. On 11/27/85 at approximately 0655 CST, this leakage remained greater than the above referenced limit, thus the reactor mode switch was placed in the shutdown position to comply with Technical Specifications Section 3.6.G.3.a. On 11/27/85 at approximately 1900 CST, the unidentified leakage was reduced to below the above referenced limit and the LCO was terminated. However, the reactor shutdown was continued so that a scheduled refueling outage could begin.

The unidentified leakage was caused by reactor recirculation pump suction valve 1B31-F023A packing leaks and by leaking pressure seal gaskets for the inner bonnet and hinge pins of feedwater check valve 1B21-F010A.

Valves 1B31-F023A and 1B21-F010A were repaired, functionally tested satisfactorily, and returned to service on 04/30/86 and 02/05/86, respectively.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104  
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT I	0 5 0 0 0 3 2 1	8 5	— 0 4 1	— 0 1	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 386A's) (17)

This 30 day LER is required by 10CFR 50.73(a)(2)(i)(A) due to the completion of a shutdown required by Technical Specifications Section 3.6.G.3.a because the plant could not meet the requirements of Technical Specifications Section 3.6.G.1.a.

On 11/26/85 at approximately 1500 CST, with the reactor mode switch in the run position and reactor power at 2367 MWt (approximately 97% rated power), plant personnel noted that the unidentified leakage into the Drywell floor drain (EIIIS identifier: IJ) was greater than 5 gpm (gallons per minute) when averaged over the last 24 hours. This is in excess of the limit specified by Technical Specifications Section 3.6.G.1.a. The required 4 hour LCO (limiting condition for operation) was initiated per Technical Specifications Section 3.6.G.3.a. On 11/26/85 at approximately 1830 CST, plant management decided to initiate a normal reactor shutdown and enter a scheduled refueling outage early due to the unidentified leakage exceeding the Technical Specifications limit. On 11/27/85 at approximately 0655 CST, the unidentified leakage into the Drywell remained greater than 5 gpm when averaged over the last 24 hours, thus the reactor mode switch was placed in the shutdown position by licensed personnel to comply with the Technical Specifications requirement to be in hot shutdown within 12 hours. On 11/27/85 at approximately 1900 CST, the unidentified leakage was reduced to approximately 4 gpm when averaged over the last 24 hours and the LCO was terminated. However, the reactor shutdown was continued so that the scheduled refueling outage could begin.

On 11/29/85 at approximately 1200 CST, with the reactor mode switch in shutdown, plant personnel noted that the unidentified leakage into the Drywell floor drain had increased by an amount greater than 2 gpm when averaged over the last 24 hours. This exceeds the limits specified in Technical Specifications Section 3.6.G.1.b. On 11/29/85 at approximately 2215 CST the leakage was noted as exceeding the limit of 5 gpm when averaged over the last 24 hours which is specified by Technical Specifications Section 3.6.G.1.a. (Note: When this event occurred, the reactor mode switch was already in the shutdown position due to a previous LCO from the first event). The reactor was placed in cold shutdown on 11/30/85 at 0410 CST.

The unidentified leakage into the drywell's floor drain was caused by reactor recirculation pump suction valve 1B31-F023A packing leaks and by leaking pressure seal gaskets for the inner bonnet and hinge pins of feedwater check valve 1B21-F010A.

Valve 1B31-F023A was repaired by replacing the packing for the top, middle, and lower stages. The valve's stem, glands, and stuffing boxes were inspected. Valve 1B31-F023A was functionally tested satisfactorily and returned to service on 04/30/86.

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TEXT (If more space is required, use additional NRC Form 386A's) (17)

Valve 1B21-F010A was repaired by replacing the pressure seal gaskets for the inner bonnet and hinge pins. Additionally a spacer ring for one hinge pin was replaced. The valve was functionally tested satisfactorily per the "PRIMARY CONTAINMENT PERIODIC TYPE B & C LEAKAGE TESTS" procedure (42SV-TET-001-1S) and returned to service on 02/05/86.

## Failed Component Identification:

- (a) Reactor Recirculation Pump Suction Valve (1B31-F023A);  
Manufacturer - Crane Valve Co; Model # LIST 600; EIIS code - AD.
- (b) Feedwater Check Valve (1B21-F010A); Manufacturer - Rockwell  
International; Model # 970 NY; EIIS code - SJ.

The leakage was contained per the design of the plant. The required Limiting Condition for Operation (LCO) was initiated, and the reactor mode switch was placed in the shutdown position to comply with Technical Specifications Section 3.6.G.3.a. Thus, the health and safety of the public were not adversely affected by this event.

There are no known previous similar events.

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L. T. Gucwa  
Manager Nuclear Safety  
and Licensing



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0166C

June 11, 1986

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report 50-321/1986-041, Rev. 1. This report meets the reporting requirements of 10 CFR 50.73(a)(2)(i)(A).

Sincerely,

L. T. Gucwa

LGB/lc

Enclosure

c: Georgia Power Company  
Mr. J. P. O'Reilly  
Mr. J. T. Beckham, Jr.  
Mr. H. C. Nix, Jr.  
GO-NORMS

Nuclear Regulatory Commission  
Dr. J. N. Grace, Regional Administrator  
Senior Resident Inspector

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