UNITED STATES



NUCLEAR REGULATORY COMMISSION

REGIONIV

611 RYAN PLAZA DRIVE, SUITE 400 ARLINGTON, TEXAS 76011 8064 January 28, 1998

Garry L. Randolph, Vice President and Chief Nuclear Officer Union Electric Company P.O. Box 620 Fulton, Missouri 65251

SUBJECT: CALLAWAY PLANT RADIATION PROTECTION PROGRAM PERFORMANCE

This refers to the meeting conducted in the Region IV office on January 21, 1998. This meeting was conducted at "he licensee's request to discuss the radiation protection, chemistry, and radwaste program activities. Specific topics of discussion included: 1997 performance indicators, challenges and improvement opportunities, and plans for Refueling Outage 9.

In accordance with Section 2.790 of the NRC's "Pules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter will be placed in the NRC's Public Document Room. Should you have any questions concerning this matter, we will be pleased to discuss them with you.

Sincerely,

Blaine Muncer

Blaine Murray, Chief Plant Support Branch Division of Reactor Safety

Docket No.: 50-483 License No.: NPF-30

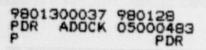
Enclosures: 1. Attendance List

2. Licensee Presentation

CC:

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Union Electric Company

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Union Electric Company

DISTRIBUTION w/Enclosures 1 and 2:

DCD (IE35) Regional Administrator Callaway Resident Inspector DRS Director DRS Deputy Director DRP Director DRS-PSB Branch Chief (DRP/B) Project Engineer (DRP/B) Branch Chief (DRP/TSS) C. A. Hackney, RSLO L. T. Ricketson, DRS/PSB MIS System RIV File

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ENCLOSURE 1

MEETING: UNION ELECTRIC - CALLAWAY PLANT

SUBJECT: Radiation Protection Activities and Performance

DATE: January 21, 1998

ATTENDANCE LIST

ORGANIZATION	POSITION TITLE
NRC	Deputy Director, Division of Reactor Safety
NRC	Chief, Plant Support Branch, Division of Reactor Safety
NPC	Project Engineer, Project Branch B, Division of Reactor Projects
NRC	Senior Radiation Specialist, Plant Support Branch, Division of Reactor Safety
Union Electric	Superintendent, Radiation Protection
Union Electric	Superintendent, Chemistry and Radwaste
	NRC NRC NRC NRC Union Electric



NRC Region IV Briefing **Callaway** Plant January 21, 1998 **Radiological Protection Areas** -Health Physics -Chemistry -Radwaste Presented by: M.S. Evans, Supt. HP R.R. Roselius, Supt. Chem/RW



Purpose

 Provide an update on Callaway Plant operations and performance
Dialogue on issues and antivities related to Radiological Protection Programs

 Provide information and answer questions on topics of interest to NRC

Agenda

1997 Performance Indicators

-HP

-Chemistry

-Radwaste

Challenges and Improvement Opportunities

-Axial Offset (AO)

-Failed Fuel

-Refuel 8 Corrective Actions

-Radwaste Processing

-Department Challenges and Improvement Opportunities

•Refuel Outage 9

-Schedule

-Goals

-Radiological Considerations

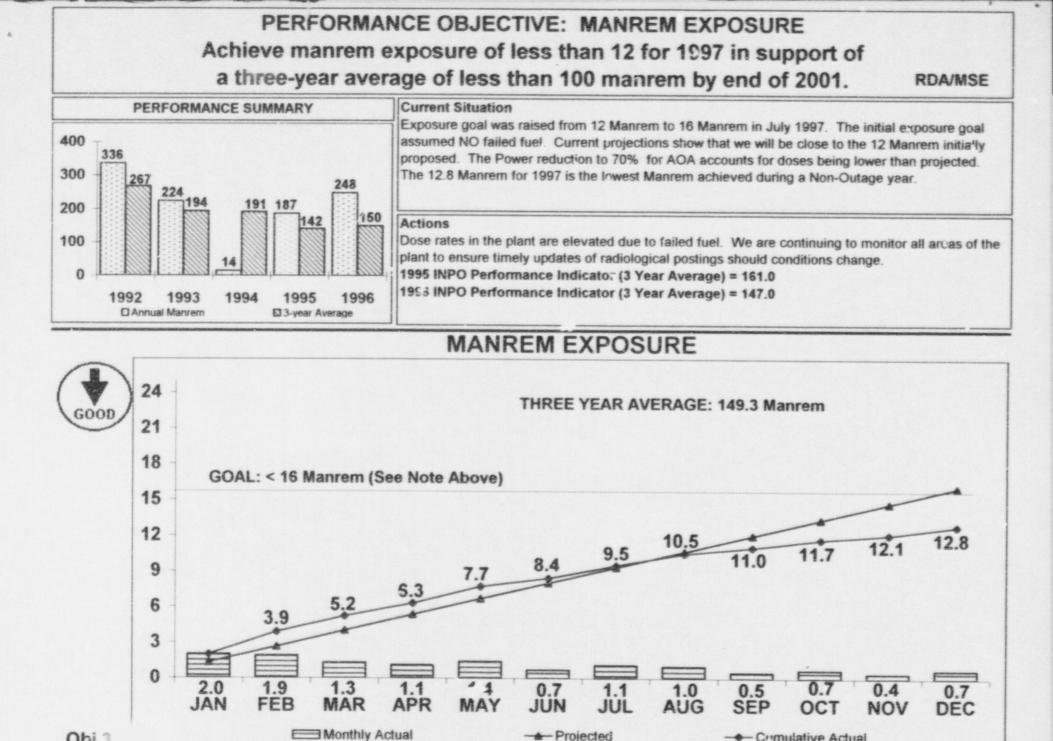
-Shutdown Chemistry



Performance Indicators

Health Physics

- Manrem Exposure
- Liquid Rad Effluents
- Gaseous Rad Effluents
- Surface Contamination
- Chemistry
 - Chemistry Index
 - Primary to Secondary Leakage
 - RCS CRUD (Co-58)
 - RCS Dose Equiv. Iodine (DEI)
- Radwaste
 - Liquid Radwaste Influents
 - DAW Generation
 - Low Level Solid Radwaste
 - Radwaste Burial Performance



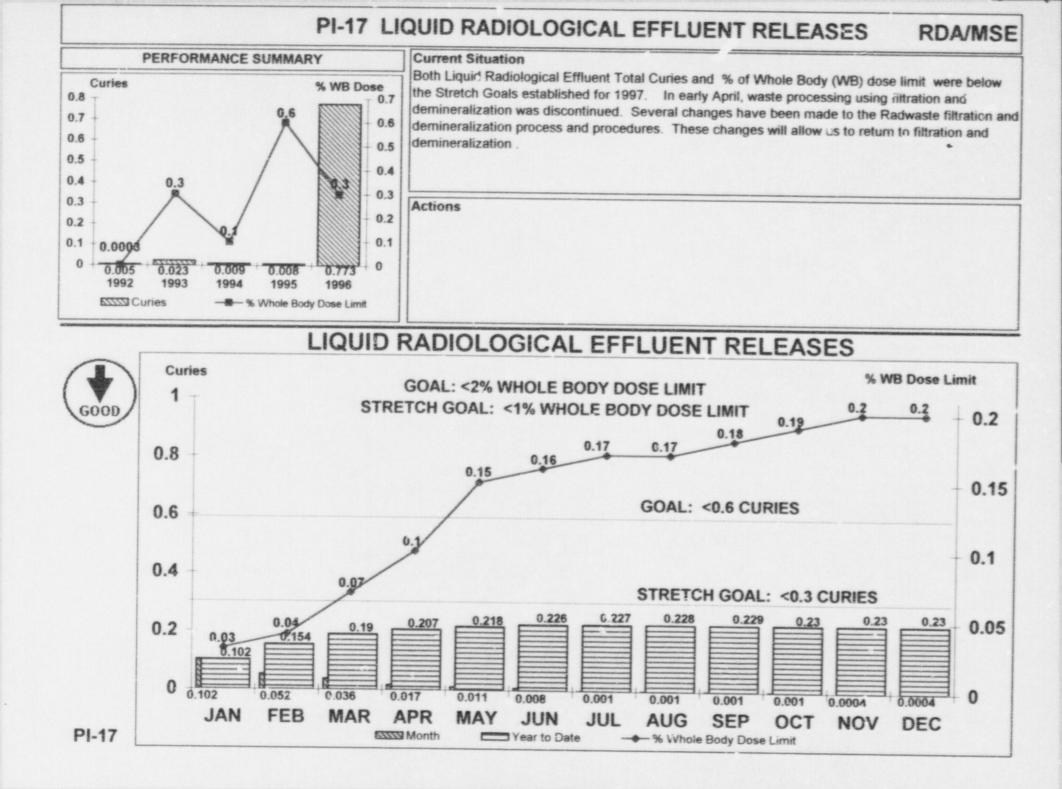
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Manrem Exposure

- 3 year average being maintained at approx. 150 Manrem
- 1997 Manrem Goal
 - Assumed NO Failed Fuel
 - Based on best performance to date of 14.2 Manrem in 1994
 - Revised in July 1997
 - Power Reduction in August 1997 reduced General Area dose rates by approx. 1/2
- Improvement Opportunities
 - AO Resolution
 - Shutdown Chemistry
 - S/G Dose
 - Approx. 25-30% of outage dose
 - · Pursuing electro-sleeving tube repair, currently under review
 - Anticipate approx. same failure rate as Refuel Outage 8
- 1998 Manrem Exposure Goal is 200 Manrem
 - 185 Manrem Refuel Outage 9 Dose
 - 15 Manrem Non-outage Dose

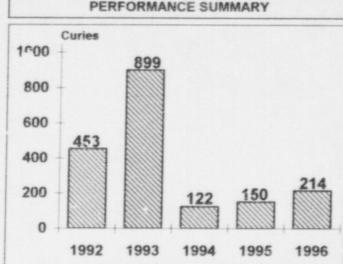


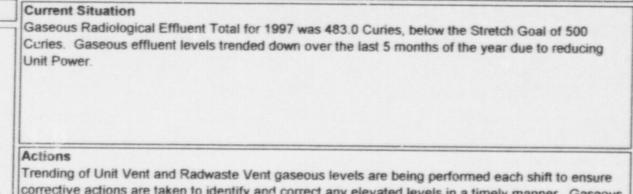


Liquid Radiological Effluents

- Goal Re-evaluated in 1996
 - Established 0.25 Curie Goal based on Industry Comparisons
 - Percent of Whole Body Limit Remained at < 1%
- Liquid Radwaste Processing
 - Moved to Filtration and Demineralization in 1996
 - Used Evaporators, Filtration, and Demineralization in 1997
 - Will use Filtration and Demineralization in 1998
- 1998 Liquid Radiological Effluents Goal is <0.25 Curies and <1% Whole Body Limit

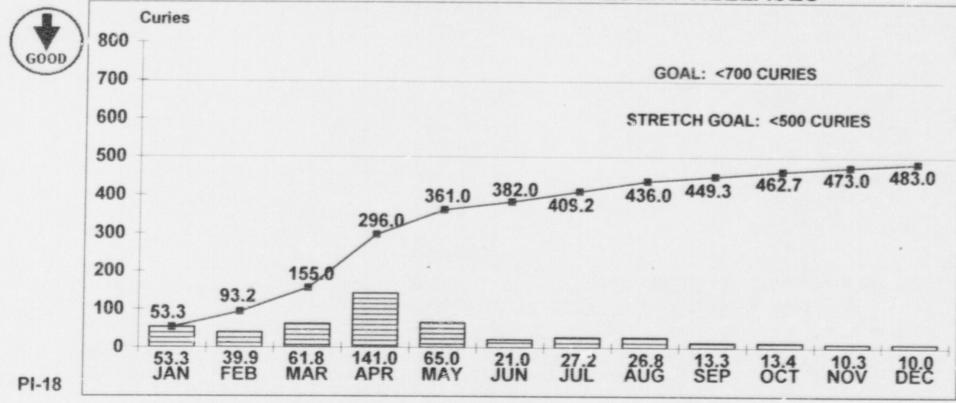
PI-18 GASEOUS RADIOLOGICAL EFFLUENT RELEASES RDA/MSE





corrective actions are taken to identify and correct any elevated levels in a timely manner. Gaseous effluents for the past 2 months have been lower than previous months this cycle due to the Power reduction to 70% for AOA.

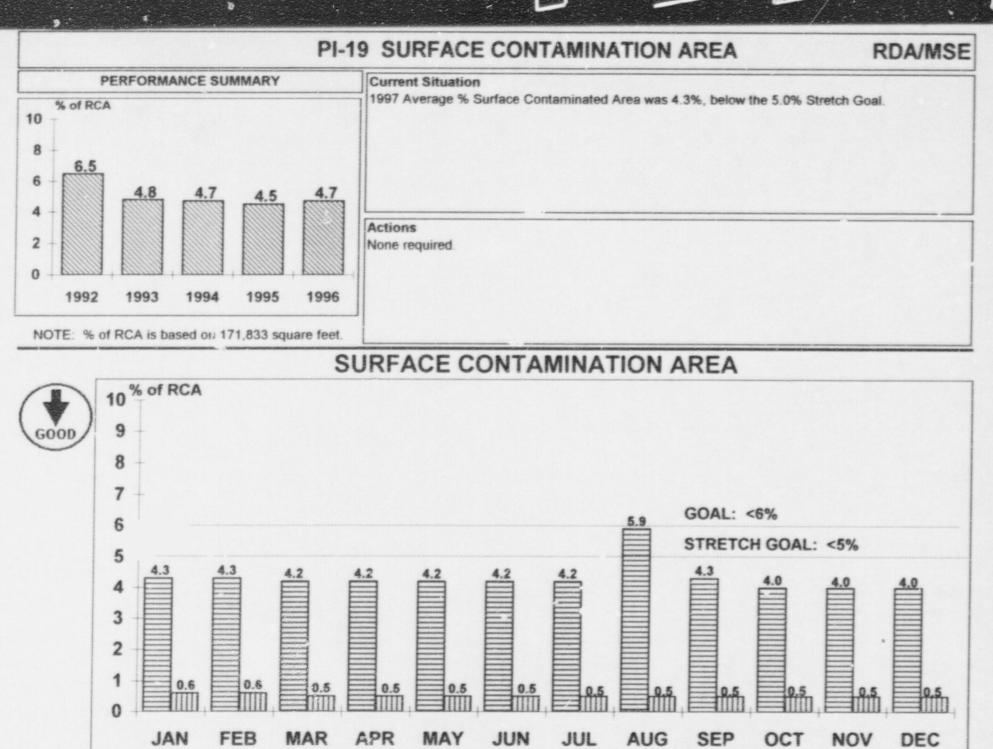
GASEOUS RADIOLOGICAL EFFLUENT RELEASES





Gaseous Radiological Effluents

- Significant Increase in 1997 due to Failed Fuel
- Power Reduction in August 1997 to 70%
- 1998 Gaseous Radiological Effluents Goal is 600 Curies



Total

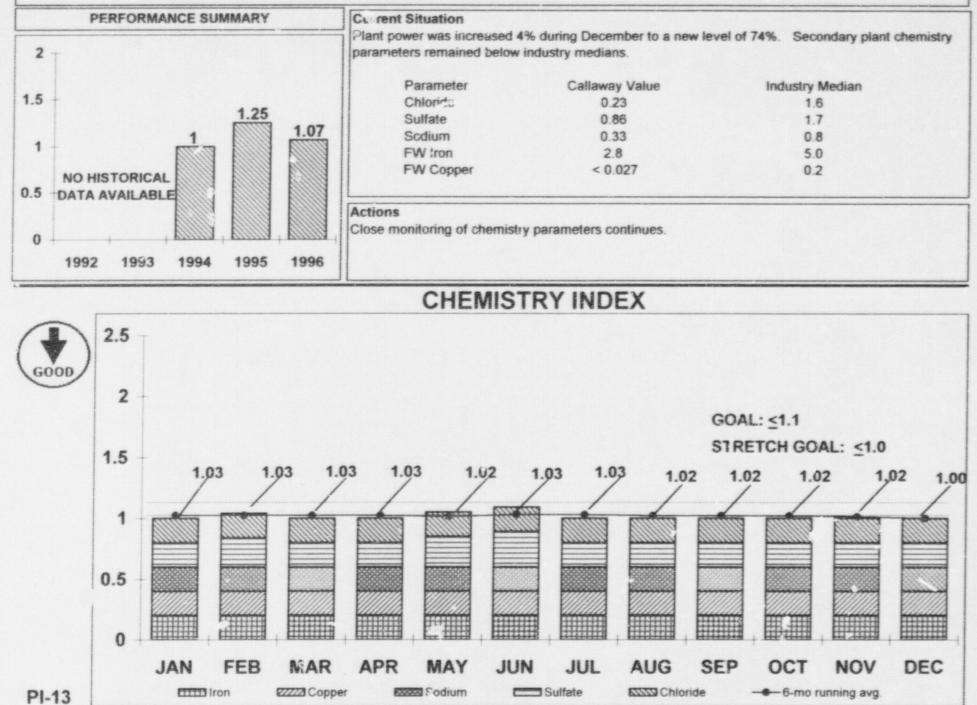
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PI-13 CHEMISTRY INDEX

RDA/ECO





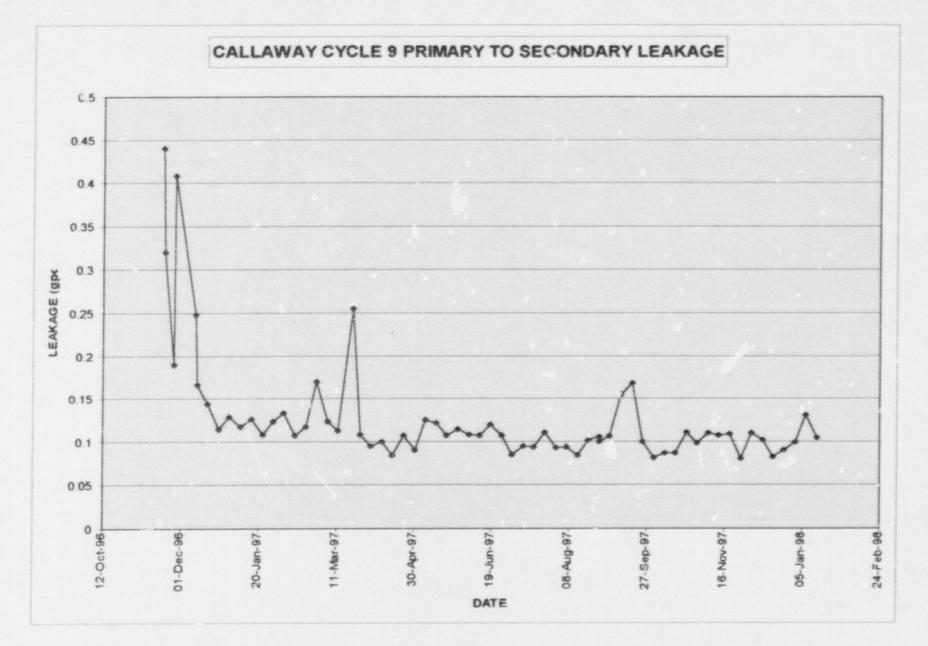
Chemistry Index

- Indicator of Secondary Chemistry performance
- Measure of how well taking care of S/G
- 1.0 is best performance achievable
- All 5 index parameters better than industry median
- 1997 average 1.01
- 1996 average 1.06

Primary to Secondary Leakage

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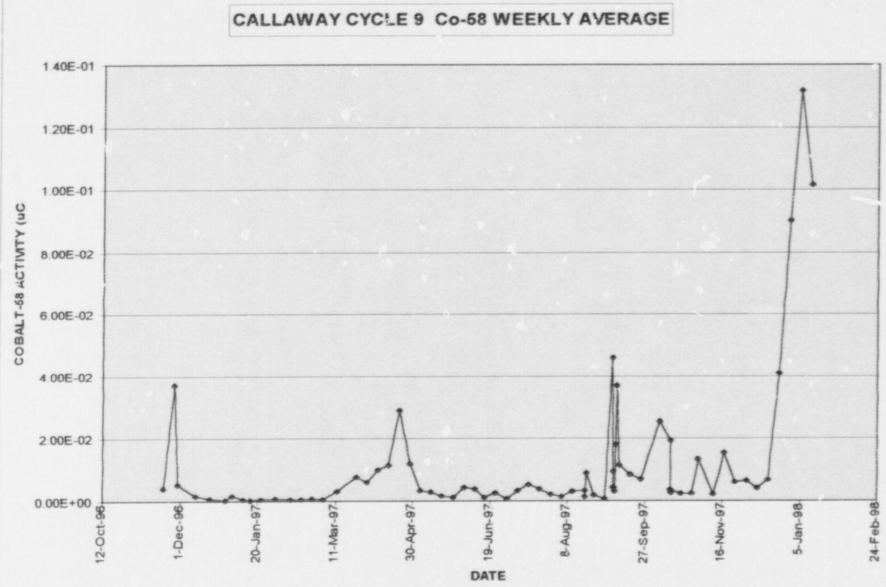


Primary to Secondary Leakage

- S/G integrity good
- Leakage dropped following RFO8
- Lowest leakage experienced for last several cycles
- N-16 monitors
- Programmatic improvements in detection and response

RCS Cobalt 58

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RCS CRUD

- Levels elevated compared to Cycle 8
- Increases with changes in Axial Offset
- Trending up w/ ammonia addition
- CVCS letdown line dose rates have not increased

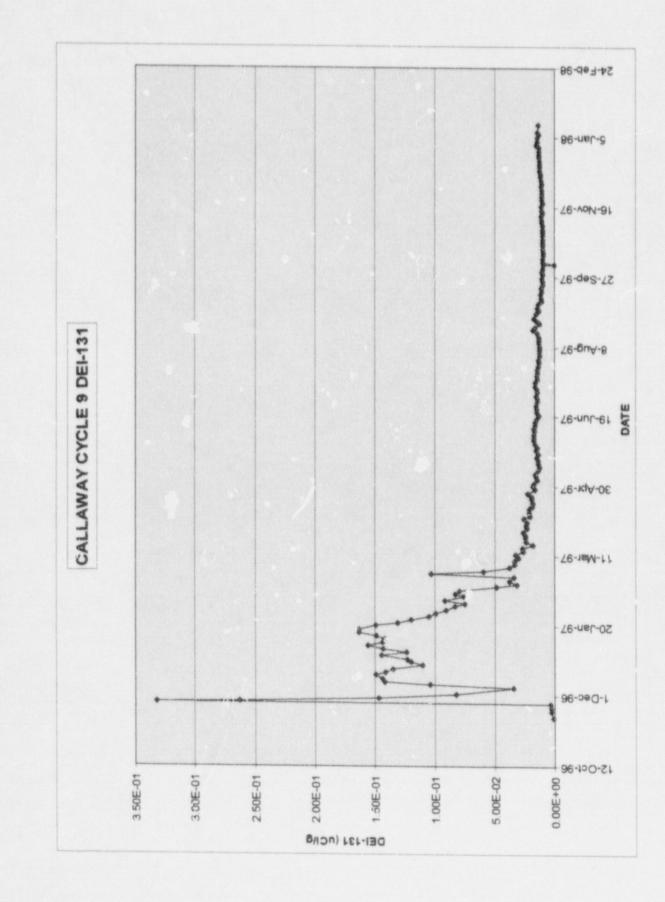
RCS Dose Equivalent Iodine

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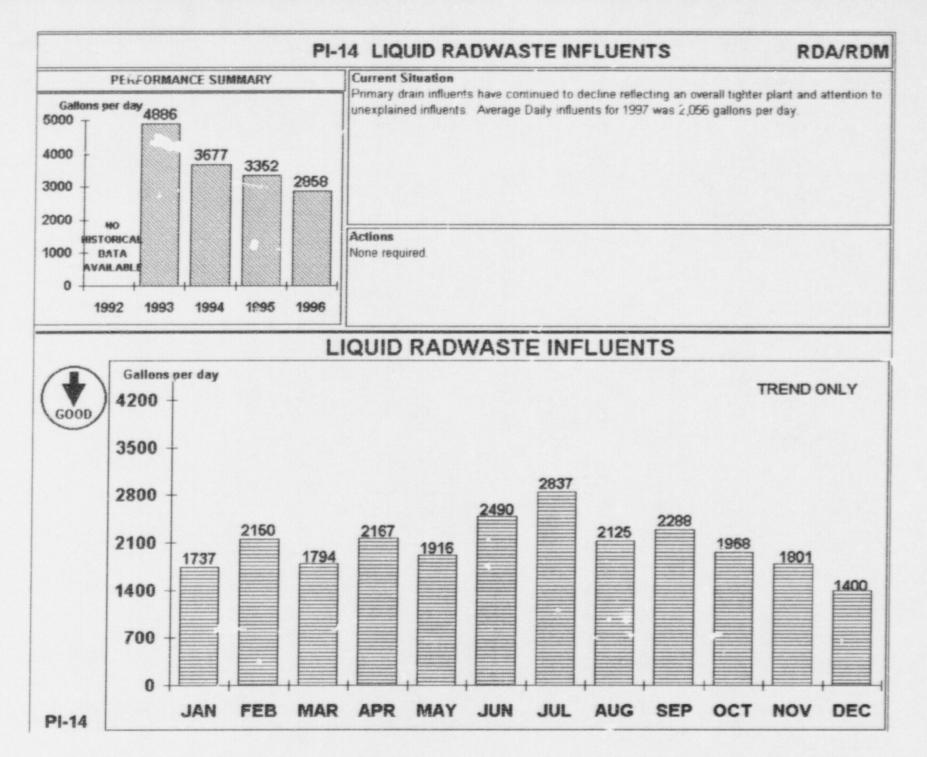
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RCS DEI-131

- Fuel defect early in cycle
- 1-3 defects
- DEI 1.3E-2 uci/ml
- Tight defect



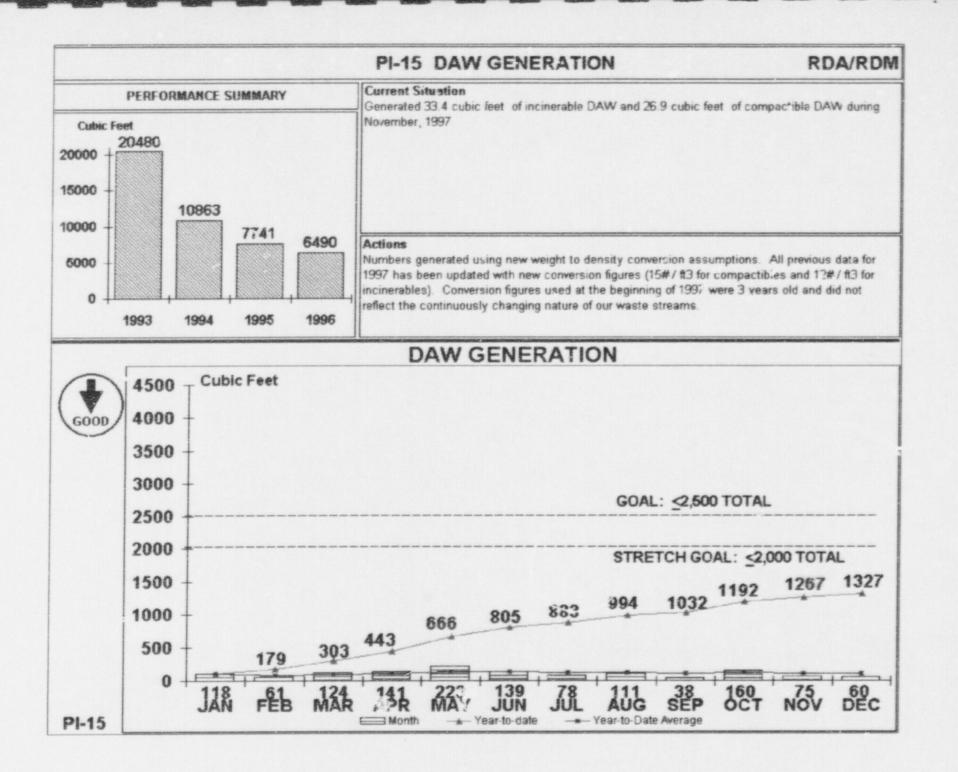
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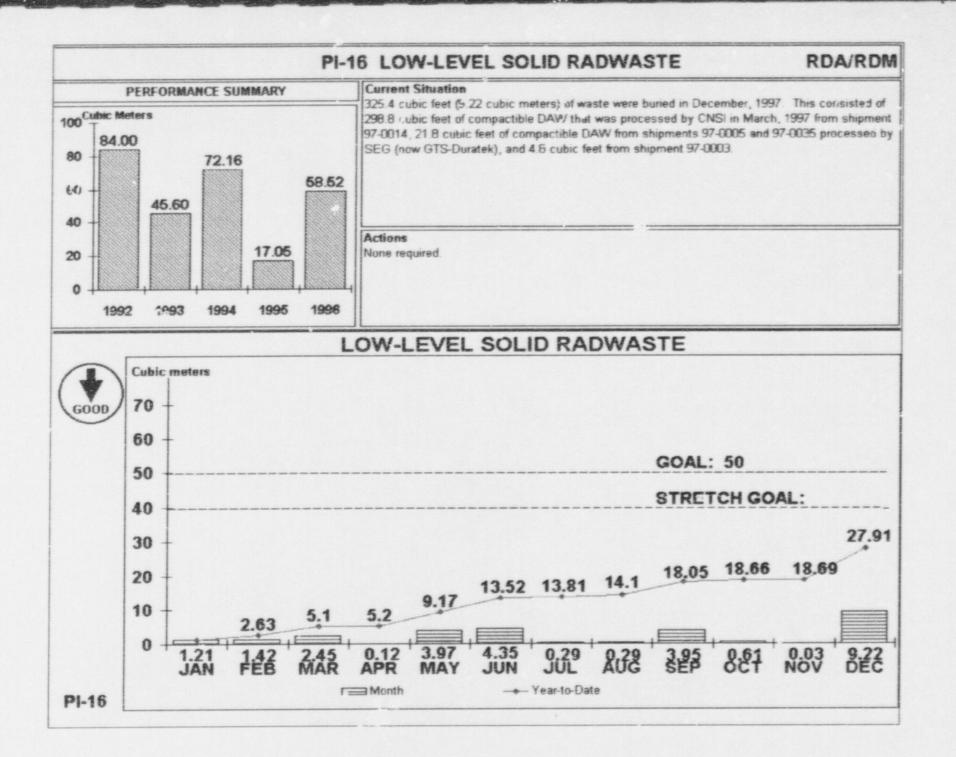
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Liquid Radwaste Influents

- Directly correlates to liquid effluent releases and solid radwaste generation
- All time low
- 28% reduction from 1996
- 57% reduction from 1993
- Reasons
 - Plant material condition
 - Coordination of draining with Operations
 - Leakage monitoring and response





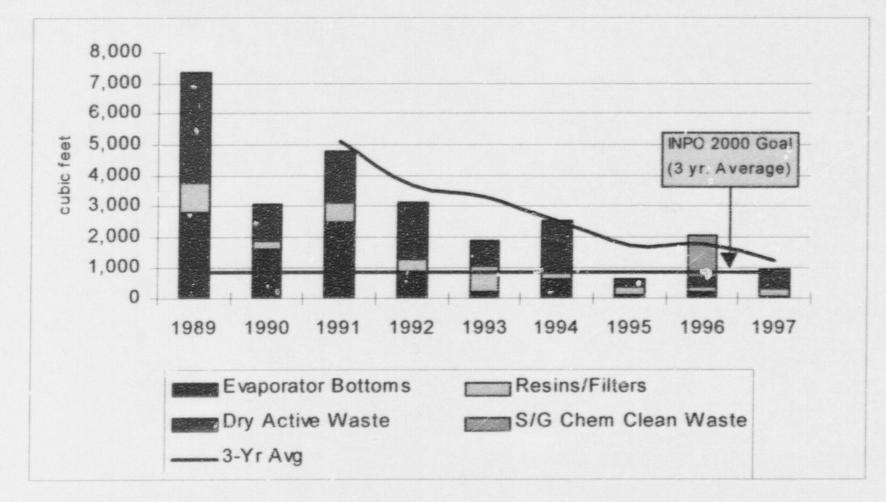
CALLAWAY BURIAL PERFORMANCE

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Challenges and Improvement Opportunities

- Axial Offset
- Fuel Defect
- Refuel 8 Corrective Actions
 - HP
 - Chemistry / Radwaste
- Radwaste Processing
 - Department Challenges and Improvement Opportunities
 - HP
 - Chemistry / Radwaste



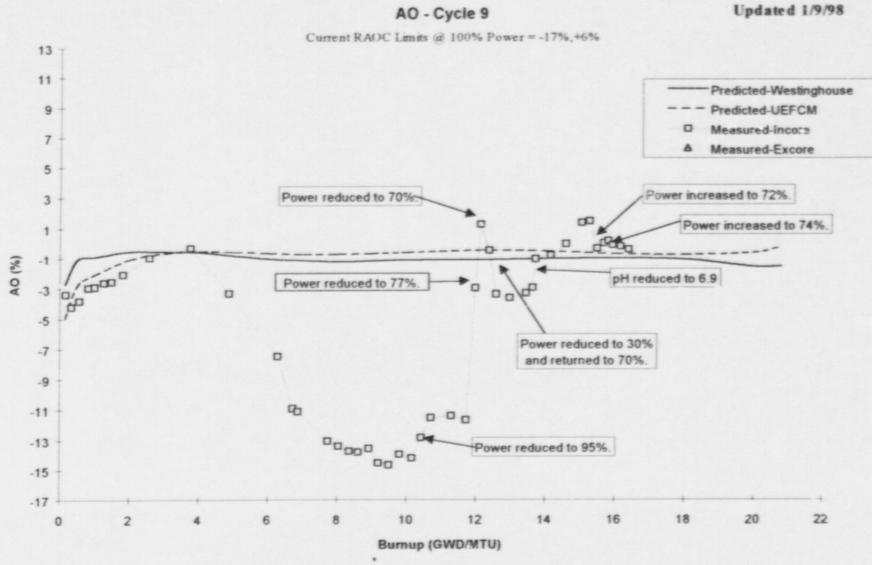
Axial Offset

- More significant than Cycle 8
- Down power to 70% due to shutdown margin concerns
- Currently at 74% w/ AO trending +
- Corrective Actions
 - Core redesigned for Cycle 10
 - RCS pH control constant 7.1 pH cycle 9 and 7.2 pH cycle 10
 - Ammonia Addition to RCS
- RFO9 Impact

Axial Offset Cycle 9

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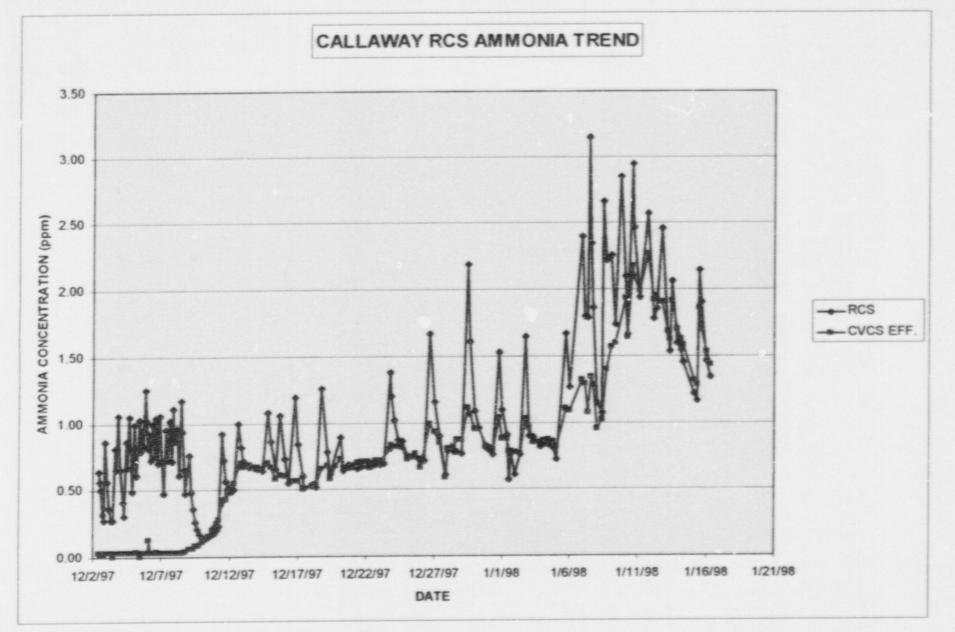




Ammonia Addition to the RCS

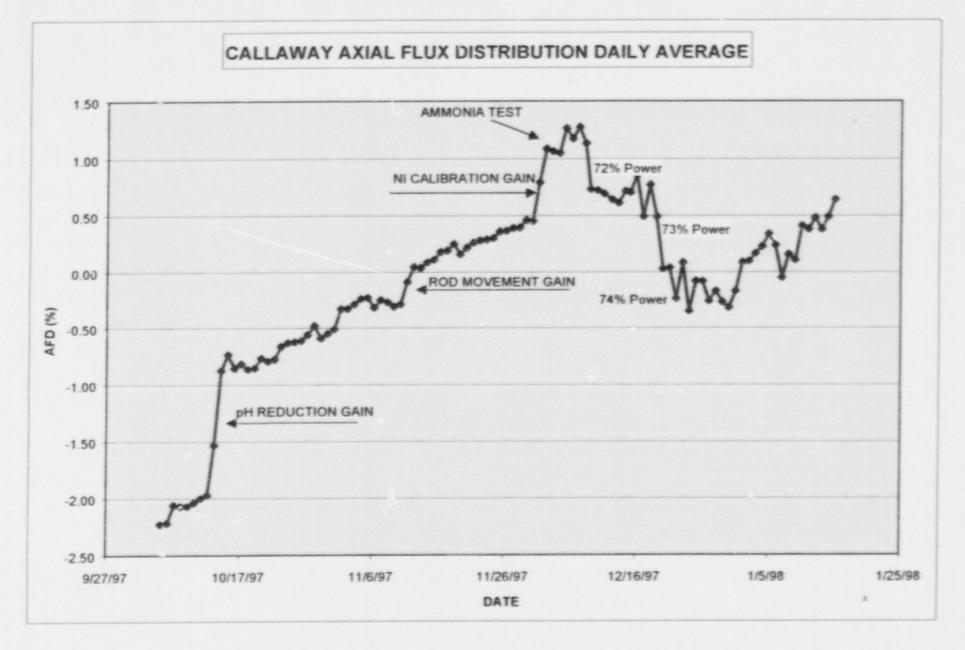
- Purpose complex with Ni rich deposits on fuel and solublize the corrosion layer resulting in AO improvement
- Technical Bases Siemens international experience
- Governed by procedure and 50.59 FSE
- Over 50 additions performed
- No CRUD Bursts or filter plugging
- RCS CRUD levels elevated

RCS Ammonia



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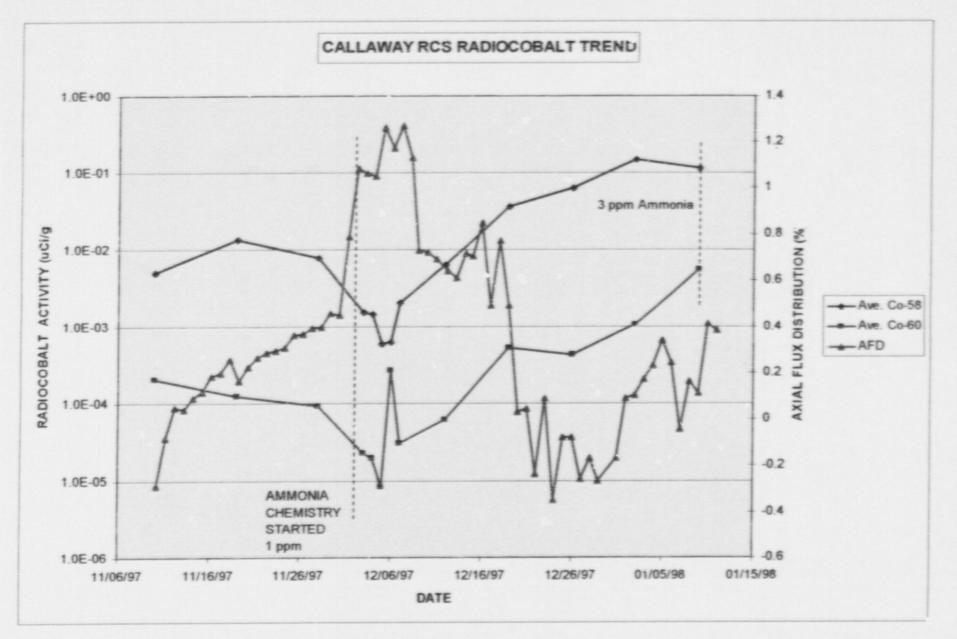
Axial Flux Distribution



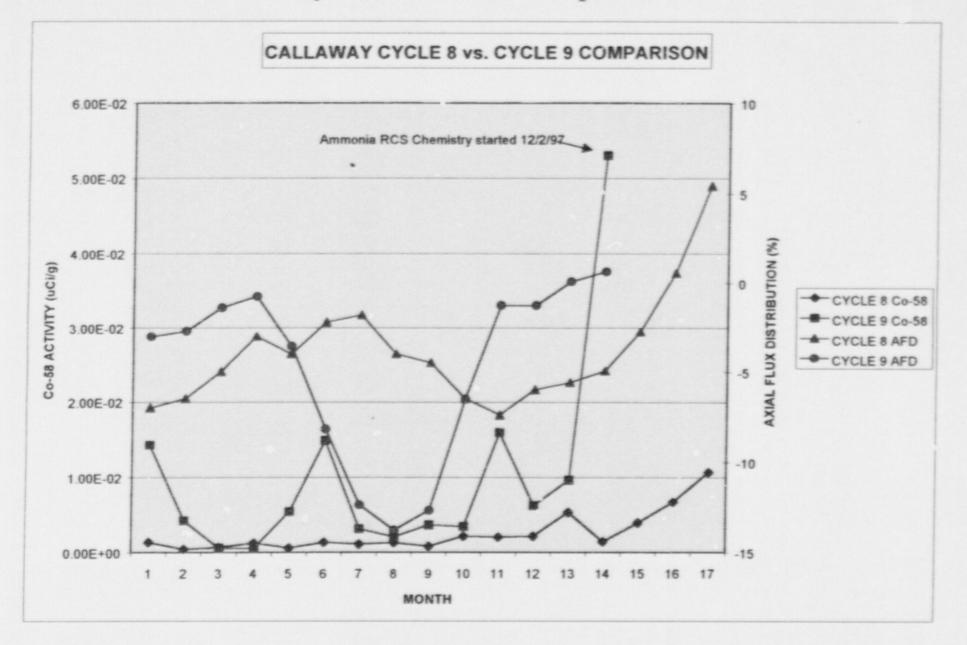
RCS Cobalt 58 and 60

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Cycle 8 and 9 Comparison





Fuel Defect

- · Failure in fresh, high flux assembly
- 20 x Fuel Reliability Index for zero defects
- RCS fission gases 0.5 -1.0 uci/g
- Fuel Failure Action Plan
- · Potential outage impact of large spike at shutdown



Refuel 8 HP Corrective Actions

- Increase Manning of Vendor Technicians due to AO impact
- Engineering Evaluation of Pressurizer Spray Flow during Cooldown is being evaluated
- Implementing additional Guidance for Cavity and Head work activities
- Prioritize Temp Shielding Installation
 - Loop Piping
 - Pressurizer Spray and Surge Lines
- Establish ALARA contact in Outage Maintenance Facility
- Re-evaluating Cavity Decon, pursuing remote vacuuming and detergent decon in lieu of diver and strippable coating
- Installation of Permanent Postings for Bioshield Entrances during Outages
- Revised procedural guidance for RCP Shipping Container Surveys and included in Training of Technicians



Chemistry RFO8 High Dose Rate Self Assessment

- Detailed Assessment conducted w/ assistance of industry experts
- Independent Review by EPRI and Peers
- Conclusions:

Root Cause - AOA Avoid modified 7.4 pH regime Shutdown Chemistry does not lower dose rate Keep H2 elevated during acid reducing conditions Extend Acid reducing time during shutdown Minimize acid reducing conditions during startup



Liquid Radwaste Processing

- · Implemented demineralization processing in July of 1996.
 - Last to use evaporators
 - Cesium selective resin
 - Install plant equipment used
- Demineralization discontinued in April 1997 due to elevated Co-58 levels
- Corrective Actions
 - Admin limit placed on activity released
 - New charcoal used for Cobalt removal
 - 0.5 micron bag filter installed
 - Implement used of additional charcoal bed for reprocessing
- Restarted Demin Processing in December 1997
- 1997 Totals

2.3 E-1 curies released

6.09 E-3 mrem (0.2% of annual limit)

HP Department Challenges and Improvement Opportunities

- FSAR Review and Procedure Revision
 - FSAR Review on-going
 - STS Implementation in March 1999
- ALARA Program Improvements
 - Outage Review Board Membership Revised
 - Post Job Review Threshold Reduced from 20 to 7.5 Manrem
 - ALARA Suggestion Program
 - Implementing site-wide simplified Suggestion Program for Refuel 9
 - 1997 SOS Suggestions 8, Requests for Resolution (Design) 10
 - 1998 YTD SOS Suggestions 2, Requests for Resolution (Design) 0
- High Radiation Area Controls
 - Signage is being revised for DHRA and DREA postings
 - Installation of Gate for HP control of Emergency Personnel Hatch

R. CALLAWAY PLANT

HP Department Challenges and Improvement Opportunities

- Self Assessment
 - ALARA Program assessment in January 1998
 - Participation in Assessments at other plants
 - Increase number of Self Assessments in HP Area
- Contamination Control
 - Conditional Release Program revised, adding additional signage
- Health Physics Facilities and Equipment
 - RCA Tool Room, Auxiliary Building 1974'- January 1998
 - Access Control Applications February 1998
 - RAM Storage Building February 1998
 - Laundry and Decontamination Facility March 1998
 - Remote Monitoring Task Team July 1998



Chemistry and Radwaste Department Challenges and Improvement Opportunities

- Self Assessments
- Housekeeping Ownership
- Sustained Radwaste performance
- FSAR Review and Procedure Revision Initiative

- Chemistry Technician Fundamentals Knowledge
- PASS Performance

	% Availability 1997
In-line Gamma Spec	94%
In-Line Boron	99%
In-line Dissolved H2	87%
Grab Sampler	89%
Overall	99%



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Refuel Outage 9

- Scope
- Schedule
- Radiological Considerations
- Shutdown Chemistry



Refuel 9 Scope

- S/G nozzle dam after fuel off load
- New refuel machine
- Replace B RCP Motor and pump internals
- 100% ECT of 4 S/G
- · Electro-sleeving tube repair
- Safety train work
- 40 Design Changes implemented



Refuel 9 Schedule

- Duration 28 days
- Shutdown to Mode 5 28 hours
- RCS Clean-up post H2O2 add 76 hours
- S/G ISI and repair 214 hours
- Fuel offload 29 hours



Refuel 9 Radiological Considerations

- High Noble Gas Levels due to Failed Fuel
- Potential Iodine Spike after Shutdown
- Potential High Particulate Levels and Associated Hot Spot and Contamination Control Concerns
- Radiation Levels may be much higher than in the past due to magnitude of AOA



Refuel 9 Shutdown Chemistry

- De-lithiate RCS
- Elevated H2
- 24 hours in acid reducing conditions
- Chem degas and H2O2 addition
- Clean-up prior to securing RCPs
- Flood up target 0.05 uci/cc Co-58 and 0.01 uci/cc I-131
- Lessons Learned from other plants