CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT
362 INJUN HOLLOW ROAD • EAST HAMPTON, CT 06424-3099

November 15, 1997
Re: Technical Specification 6.9.1-8
Docket No. 50-213
CY-970-122

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

In accordance with reporting requirements of Technical Specification 6.9.1.8, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 97-10 covering operations for the period October 1, 1997 to October 31, 1997 is hereby forwarded.

Very truly yours,

Russell A. Mellor
Director of Site Operations
and Decommissioning
Haddam Neck Station

RAM/bom
cc: (1) Regional Administrator, Region 1
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

(2) William J. Raymond
Sr. Resident Inspector
Connecticut Yankee

IE24
1/1

Connecticut Yankee Atomic Power Company

Haddam Neck Plant

Haddam, Connecticut

Monthly Operating Report No. 97-10

For The Month of

October 1997

Plant Operating Summary

The Haddam Neck Plant has been shut down since July 22, 1996 and on December 4, 1996, the Connecticut Yankee Atomic Power Company (CYAPCO) Board of Directors voted to permanently and immediately cease operations prior to expiration of its license.

The removal of fuel from the reactor vessel started on November 13, 1996. As of November 15, 1996, all fuel assemblies were permanently removed from the reactor and placed in the spent fuel pool for temporary storage.

On December 5, 1996, pursuant to 10CFR50.82(a)(1)(i) and 10CFR50.82(a)(1)(ii), CYAPCO provided docketed certification to the NRC that it has decided to permanently cease operations at the Haddam Neck Plant and that fuel has been permanently removed from the reactor.

CYAPCO submited the Post-Shutdown Decommissioning Activities Report (PSDAR) and the site-specific decommissioning cost estimate in accordance with 10CFR50.82 on August 22, 1997.

On October 27, 1997 the NRC held an infomational meeting at the Haddam Killingworth High School in Higganum, CT to solicit public comments on the PSDAR.

NRC OPERATING STATUS REPORT

Haddam Neck

1. Docket: 50-213
2. Reporting Period: 10/97 Outage + On-line Hours: 745.0 + 0.0 = 745.0
3. Utility Contact: K.W. Emmons (203) 267-3654
4. Licensed Thermal Power (MWt): 1825
5. Nameplate Rating (Gross MWe): 667 x 0.9 = 600.3
6. Design Electrical Rating (Net MWe): 582
7. Maximum Dependable Capacity (Gross MWe): 586.9
8. Maximum Dependable Capacity (Net MWe): 560.1
9. If changes occur above since last report, reasons are: NONE
10. Power level to which restricted, if any (Net MWe): N/A
11. Reasons for restriction, if any: N/A

	MONTH	YEAR-TO-DATE	CUMULATIVE
12. Report period hours:	745.0	7,296.0	261,528.0
13. Hours reactor critical:	0.0	0.0	199,493.6
14. Reactor reserve shutdown hours:	0.0	0.0	1,785.0
15. Hours generator on-line:	0.0	0.0	193,539.3
16. Unit reserve shutdown hours:	745.0	7,295.0	8,341.0
17. Gross thermal energy generated (MWtH):	0.0	0.0	335,551,124.0 *
18. Gross electrical energy generated (MWeH):	0.0	0.0	110,009,804.0 *
19. Net electrical energy generated (MWeH):	-1,314.4	-13,832.3	104,500,691.3 *
20. Unit service factor:	0.0	0.0	74.0
21. Unit availability factor:	99.9	100.0	77.3
22. Unit capacity factor using MDC net:	0.0	0.0	72.1
23. Unit capacity factor using DER net:	0.0	0.0	68.7
24. Unit forced outage rate:	0.0	0.0	6.3
25. Forced outage hours:	0.0	0.0	12,370.0

26. Shutdowns scheduled over next 6 months (type,date, duration): NONE

27. If currently shutdown, estimated startup date: N/A

* Cumulative values from the first criticality (07/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).

CUMULATIVE REPORT

10/97

	CUMULATIVE
1. Report period hours:	265,392.0
2. Hours generator on-line:	195,083.9
3. Hours reactor critical:	201,702.3
4. On-line net electrical energy generated (MWeH):	104,745,254.2
5. Off-line station service electrical energy utilized (MWeH):	269,735.5
6. Transformer losses (MWeH):	276,153.1
7. Effective full power days:	7,734.1
8. Effective full power hours:	185,617.3
9. Core burn-up (MWtD/MTU):	218,701.8
10. Number of reactor shutdowns:	307

Note: all values are cumulative from the first criticality (0104 hrs, 07/24/67).

AVERAGE DAILY POWER LEVEL

Docket No: 50-213
Unit: Haddam Neck
Date: 11/12/97
Completed by: K.W. Emmons
Telephone: (860)447-1791 X6572

Month: Oct-97

DAY	AVERAGE POWER LEVEL (MWe-Net)	DAY	AVERAGE POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-213
UNIT NAME Haddam Neck
DATE 11/01/97
COMPLETED BY K.W. Emmons
TELEPHONE (860) 447-1791 X6572

REPORT MONTH October 1997

No.	Date	Type1	Duration (Hours)	Reason2	Method of Shutting Down Reactor3	License Event Report #	System Code4	Component Code5	Cause & Corrective Action to Prevent Recurrence
96-05	07/22/96	S	745	F	4	N/A	N/A		

1F: Forced
S: Scheduled

2Reason
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & License Examination
F - Administrative
G - Operational Error (Explain)
(Duration = 0)

3Method
1 - Manual
2 - Manual Scram
3 - Automatic Scram
4 - Continued from Previous Month
5 - Power Reduction
6 - Other (Explain)

4IEEE Standard 805-1984, "Recommended Practices for System Identification in Nuclear Power Plants and Related Facilities"

5IEEE Standard 805A-1983, "Recommended Practices

H - Other (Explain)

for Unique identification in

Power Plants and Related

Facilities - Component

Function Identifiers"

REFUELING INFORMATION REQUEST
October 1997

1. Name of the facility: Haddam Neck

2. Scheduled date for next refueling outage: n/a (decommissioning)

3. Scheduled date for restart following refueling: n/a (decommissioning)

4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
   n/a

   (b) If answer is yes, what, in general, will these be?
   n/a

   (c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine if any unreviewed safety questions are associated with the core reload?
   n/a

   (d) If no such review has taken place, when is it scheduled?
   n/a

5. Scheduled date(s) for submitting licensing action and supporting information:
   n/a

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
   n/a

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:
   In Core: (a) 0 In Spent Fuel Pool: (b) 1019

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:
   Present storage capacity: 1480

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming present license capacity:
   n/a