Tech. Spec. 6.9.A.2

Boston Edison
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360-5599

Nancy L. Desmond
Regulatory Relations Group Manager

January 14, 1998
BECo Ltr. 2.98.003

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50.293
License No. DPR-35

December 1997 Monthly Report

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the operational status summary for Pilgrim Nuclear Power Station is provided in the attachment for your information and planning. Should you have any questions concerning this report, please contact me directly.

Joe Keene for
N. L. Desmond

RLC/dcg/decmonth
Attachment: December 1997 Monthly Report

cc: Mr. Hubert Miller
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

IE24

Attachment

OPERATING DATA REPORT

DOCKET NO. 50-293
NAME: Pilgrim
COMPLETED BY: R. L. Cannon
TELEPHONE: (508) 830-8321
REPORT MONTH: December 1997

OPERATING STATUS	NOTES
1. Unit Name	Pilgrim I
2. Reporting Period	December 1997
3. Licensed Thermal Power (MWt)	1998
4. Nameplate Rating (Gross MWe)	678
5. Design Electrical Rating (Net MWe)	655
6. Maximum Dependable Capacity (Gross MWe)	696
7. Maximum Dependable Capacity (Net MWe)	670

8. If Changes Occur in Capacity Ratings (Item Numbers 3 through 7) Since Last Report, Give Reasons:
No Changes

9. Power Level To Which Restricted, If Any (Net MWe): None

10. Reasons For Restrictions, If Any: N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period	744.0	8760.0	219696.0
12. Hours Reactor Critical	637.1	7068.0	141714.9
13. Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14. Hours Generator On-Line	572.1	6841.7	137011.5
15. Hours Unit Reserve Shutdown	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1030708.0	13135510.0	245611440.0
17. Gross Electrical Energy Generated (MWH)	356590.0	4481870.0	83289484.0
18. Net Electrical Energy Generated (MWH)	342749.0	4310431.0	80078394.0
19. Unit Service Factor	76.9	78.1	62.4
20. Unit Availability Factor	76.9	78.1	62.4
21. Unit Capacity Factor (Using MDC Net)	68.8	73.4	54.4
22. Unit Capacity Factor (Using DER Net)	70.3	75.1	55.6
23. Unit Forced Outage Rate	23.1	11.7	11.5

24. Shutdowns, Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None

If Shutdown At End Of Report Period, Estimate Date Of Start-Up: Unit Operating

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OPERATION SUMMARY

The plant entered December in a shutdown condition performing maintenance on main steam isolation valves (MSIVs) AO-203-1C and AO-203-2B. On 12/3, at 2217 hours, the generator was synchronized to the grid. On 12/5, at 0144 hours, the plant commenced reducing power from approximately 35% core thermal power (CTP) in response to a nitrogen leak detected in the drywell. On 12/5, at 0920 hours, the plant was taken off line to facilitate isolation of a nitrogen leak in the drywell. On 12/5, at 2252 hours, the generator was synchronized to the grid. On 12/6, at 0908 hours, the plant experienced a reactor scram in response to a reactor high water level signal resulting from problems with one of the two feedwater regulating valves. On 12/10, at 0103 hours, the generator was synchronized to the grid. On 12/11, at 1130 hours, the plant attained 100% CTP where it was maintained until 12/12. On 12/12, at approximately 1056 hours, the plant reduced power in response to a recirculating pump trip. On 12/13, at 1638 hours, the plant attained 100% CTP where it was essentially maintained through the end of the reporting period. One safety relief valve was lifted during this period for post work testing. Other maintenance activities were performed in accordance with the forced outage work schedule.

UNIT SHUTDOWNS

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	CAUSE/ CORRECTIVE ACTION/COMMENTS
1 con't	971201	F	84.0	A	1	Reactor was manually shutdown when main steam isolation valves AO-203-1C and AO-203-2B did not indicate fully closed during surveillance testing. The details regarding this shutdown are documented in License Event Report 97-025-00.
2	971206	F	87.9	A	3	Automatic scram due to high reactor water level during power ascension from the forced shutdown of November 23, 1997. The high water level resulted from failure of the "A" feedwater regulating valve. The details regarding this shutdown are documented in License Event Report 97-026-00.

1

F - Forced
S - Scheduled

2

A - Equip Failure
B - Main or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & License Examination
F - Admin
G - Operation Error
H - Other

3

1 - Manual
2 - Manual Scram
3 - Auto Scram
4 - Continuation
5 - Other