

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT I										DOCKET NUMBER (2) 0 5 0 0 0 3 2 1										PAGE (3) 1 OF 3			
TITLE (4) DISCONNECTED JUMPER CAUSES CLOSURE OF RWCU PRIMARY CONTAINMENT ISOLATION VALVE																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)											
0	4	2	5	8	6	8	6	0	2	0	0	5	2	3	8	6	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8. (Check one or more of the following) (11)																					
4		20.402(b)				20.406(e)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)									
POWER LEVEL (10)		0 0 1 0				20.406(a)(1)(i)				50.38(e)(1)				50.73(a)(2)(v)				73.71(c)					
		20.406(a)(1)(ii)				50.38(e)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 388A)									
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME Raymond D. Baker, Nuclear Licensing Manager - Hatch												TELEPHONE NUMBER 4 1 0 1 4 5 1 2 6 1 - 1 7 0 1 1 6											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS													
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 04/25/86 at approximately 1445 CST with the Unit in cold shutdown for a refueling outage, plant personnel were performing a functional test and calibration procedure (FT&C) when the Reactor Water Cleanup (RWCU) system's inboard and outboard primary containment isolation valves (2G31-F001 and 2G31-F004, respectively) isolated (i.e., closed) on a "high leakage flow" signal.

An investigation found that a temporary jumper, installed as part of the FT&C procedure, had become disconnected. The jumper prevents isolation of valves 2G31-F001 and 2G31-F004 when portions of the logic are actuated during procedure performance.

No cause could be determined for the disconnection of the jumper. The isolation was reset, 2G31-F001 and 2G31-F004 were opened, and RWCU was returned to service on 04/25/86 at approximately 1550 CST.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) EDWIN I. HATCH, UNIT I	DOCKET NUMBER (2) 0 5 0 0 0 3 2 1 8 6 — 0 2 0 — 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 368A's) (17)

A. REQUIREMENT FOR REPORT

This LER is required per 10CFR 50.73(a)(2)(iv) because an unplanned actuation of an engineered safety feature (closure of primary containment isolation valves 2G31-F001 and 2G31-F004) occurred.

B. UNIT(s) STATUS AT TIME OF EVENT

On 04/25/86 at approximately 1445 CST, the Unit was in cold shutdown for a refueling outage.

C. DESCRIPTION OF EVENT

On 04/25/86 at approximately 1445 CST, plant personnel were performing the "RWC SYSTEM DIFF FLOW INSTRUMENT FT&C" procedure (57SV-G31-002-1) when the Reactor Water Cleanup (RWC) system's inboard and outboard primary containment isolation valves (2G31-F001 and 2G31-F004, respectively) isolated (i.e., closed) on a "high leakage flow" signal.

D. CAUSE OF EVENT

An investigation found that a temporary jumper from terminals CC-77 and CC-78 (as required per 57SV-G31-002-1) had become disconnected at terminal CC-78. The jumper prevents isolation of valves 2G31-F001 and 2G31-F004 when sections of the logic are actuated during procedure performance. When the jumper became disconnected, it allowed the completion of the logic and the valves isolated. This caused the "high leakage flow" signal.

The cause of the jumper becoming disconnected is unknown. Other plant personnel were working in the panel. However, it could not be confirmed that they caused the jumper to become disconnected.

E. ANALYSIS OF EVENT

The RWC primary containment isolation valves (2G31-F001 and 2G31-F004) responded to the high leakage flow isolation signal as required had the signal been indicative of an actual high leakage condition. This event had no adverse effect on plant operation. The health and safety of the public were not affected by this event.

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EDWIN I. HATCH, UNIT I

0 5 0 0 0 3 2 1 8 6 - 0 2 0 - 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

F. CORRECTIVE ACTIONS

The isolation was reset, 2G31-F001 and 2G31-F004 were opened, and RWCU was returned to service on 04/25/86 at approximately 1550 CST. Procedure 57SV-G31-002-1 was successfully completed on 04/26/86. This is considered to be an isolated event, and no further corrective action is required.

G. ADDITIONAL INFORMATION

1. FAILED COMPONENT(S) IDENTIFICATION

No components failed during this event.

2. PREVIOUS SIMILAR EVENTS

There have been no past similar events.

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L. T. Gucwa
Manager Nuclear Safety
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May 23, 1986

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report 50-321/1986-020. This report meets the reporting requirements of 10 CFR 50.73(a)(2)(iv).

Sincerely,

L. T. Gucwa

EBS/lc

Enclosure

c: Georgia Power Company
Mr. J. P. O'Reilly
Mr. J. T. Beckham, Jr.
Mr. H. C. Nix, Jr.
GO-NORMS

Nuclear Regulatory Commission
Dr. J. N. Grace, Regional Administrator
Senior Resident Inspector

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