



SVP-97-320

December 30, 1997

U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Subject: Quad Cities Nuclear Power Station Units 1 and 2
Reactor Vessel Closure Studs
NRC Docket No. 50-254 and 50-265

Reference: (a) L.W. Pearce (ComEd) letter SVP-97-280 to USNRC, Response
to NRC Inspection Report Numbers 50-254/97018 and
50-265/97018, dated December 10, 1997

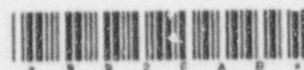
The purpose of this letter is to provide the results of an evaluation to address the fatigue cumulative usage factor (CUF) of reactor pressure vessel (RPV) closure studs as discussed in Reference (a).

An evaluation was performed (calculation #QDC-0200-S-0536 Revision 0) using the same methodology applied in the 1990 evaluation, to determine the accuracy of the prediction of reaching a CUF of 1.0 in 1998. The original evaluation was based on predicted cycles. This evaluation is based on actual cycles experienced as of November 1997. We then determined cycles per year average for the last five years (e.g. five cycles per year) to predict when a CUF of 1.0 would be attained.

An additional twenty (20) thermal cycles (e.g. heatup/cooldown, etc.) coupled with two (2) cycles of boltup/pressure test/unbolt per Unit, was used to conservatively determine that the RPV studs will attain a CUF of 1.0 after an additional four (4) years of operation. Therefore, the RPV studs are predicted to reach a CUF of 1.0 in the year 2002. The previous evaluation had over predicted the number of operating cycles and therefore, underestimated the time to attain a CUF of 1.0 (e.g. 1998).

The next planned action item as committed to in Reference (a), is to revise the UFSAR to reflect the results of this evaluation and to submit a plan detailing the necessary long term actions as previously discussed in Reference (a).

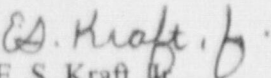
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If you have any questions concerning this letter, please contact Mr. Charles Peterson, Regulatory Affairs Manager, at (309) 654-2241, extension 3609.

Respectfully,


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Site Vice President
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