

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Peach Bottom Atomic Power Station - Unit 3DOCKET NUMBER (2)
05000278PAGE (3)
1 OF 04

TITLE (4)

Exceeding Local Leak Rate Test Allowable Limit

EVENT DATE (5)
MONTH DAY YEAR
09 30 85

LER NUMBER (6)

SEQUENTIAL NUMBER

REVISION NUMBER

REPORT DATE (7)

MONTH DAY YEAR

01 15 85

OTHER FACILITIES INVOLVED (8)

FACILITY NAME

DOCKET NUMBER

05000278

05000278

OPERATING MODE (9)

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 1. (Check one or more of the following) (11)

POWER LEVEL (10)

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED ONE NO. 3150-0194

EXPIRES 8/31/86

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 7 8	LER NUMBER (5)			PAGE (3)		
		YEAR 8 5	SEQUENTIAL NUMBER 0 1 5	REVISION NUMBER 0 1 0		OF	0 4

TEXT (If more space is required, use add-on label NRC Form 366a (1))

Description of the Event:

On September 30, 1985 it was determined that the combined leak rate of all penetrations and valves subject to Type B and C local leak rate tests had exceeded the allowable limit of 0.6 La. La is defined as 0.5%/day of the contained volume of gas in containment at peak accident pressure. For Peach Bottom Unit 3 0.6 La is 71,186 scc/min at 49.1 psig.

The following valves were identified as having excessive leak rates: 'B' instrument nitrogen return-to-drywell check valve, oxygen analyzer return-to-drywell check valve, high pressure coolant injection system turbine exhaust check valve (3-23-65), residual heat removal system torus cooling valves MO-3-10-39A and MO-3-10-34B, reactor water cleanup system return-to-reactor valve MO-3-12-68, 'A' ADS backup nitrogen solenoid valve SV-9130A, core spray system injection valve MO-3-14-11B, and feedwater check valve 3-6-28A.

The EIIIS codes for the affected systems are as follows:
Instrument Nitrogen - LK, Oxygen Analyzer - IK, High Pressure Coolant Injection - BJ, Residual Heat Removal - BO, Reactor Water Cleanup - CE, Core Spray - BM, and Feedwater - SJ.

Consequences of the Event:

For each valve listed, there is a redundant series isolation valve. The local leak rate test results of the redundant isolation valves were acceptable; therefore, containment integrity would have been maintained in the event of an accident.

Cause of the Event:

The cause of the excessive valve leakages will be investigated when the valves are disassembled and inspected. The results of these inspections will be submitted in a follow-up LER report within 30 days after the end of the current refueling outage.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3190-0104

EXPIRES 8/31/86

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 7 8 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 1 5	0 0	0 3	OF	0 4

TEXT (if more space is required, use additional NRC Form 366a (1))

Corrective Actions:

Unit 3 is presently in a refueling outage. All valves with unacceptable leak rates will be repaired and retested prior to returning Unit 3 to operation. These corrective actions will ensure that the total combined leakage rate for all Type B and C tests will be below 0.6 La.

Previous Similar Occurrences:

Peach Bottom LER's 2-85-01 and 3-85-05.

PHILADELPHIA ELECTRIC COMPANY

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PHILADELPHIA, PA. 19101

(215) 841-4000

October 29, 1985

Docket No. 50-278

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Peach Bottom Atomic Power Station - Unit 3

This LER addresses exceeding the total leakage rate limit for Type B and C tests.

Reference:	Docket No. 50-278
Report Number:	3-85-15
Revision Number:	00
Event Date:	September 30, 1985
Report Date:	October 29, 1985
Facility:	Peach Bottom Atomic Power Station RD 1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
T. P. Johnson, NRC Resident Inspector

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