NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Peach Bottom Atomic Power Station - Unit 3	0 5 0 0 0 2 7 8	1 OF 0 4

TITLE (4)
Exceeding Local Leak Rate Test Allowable Limit

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 9	3 0	8 5	8 5	-	0 1 5	- 0 0	1 0	2 9	8 5		0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	0 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
W. C. Birely, Senior Engineer, Licensing Section	AREA CODE 2 1 5 8 4 1 - 5 0 4 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	L K	I S V	N 4 2 5	Y		X	B J	I I S V	A 1 5 8	Y	
X	I K	I S V	V 1 3 9	Y		X	B O	I I S V	W 1 0 3 0	Y	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	*			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

*Please See "Cause Section"

Abstract: 3-85-15

On September 30, 1985, the containment leak test program identified that the total combined leakage of the Type B and C tests exceeded the allowable leak rate limit. Several valves tested had excessive leakage rates which contributed to the combined leakage total. These valves will be repaired and retested prior to returning Unit 3 to power operation.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A (17))

Description of the Event:

On September 30, 1985 it was determined that the combined leak rate of all penetrations and valves subject to Type B and C local leak rate tests had exceeded the allowable limit of 0.6 La. La is defined as 0.5%/day of the contained volume of gas in containment at peak accident pressure. For Peach Bottom Unit 3 0.6 La is 71,186 scc/min at 49.1 psig.

The following valves were identified as having excessive leak rates: 'B' instrument nitrogen return-to-drywell check valve, oxygen analyzer return-to-drywell check valve, high pressure coolant injection system turbine exhaust check valve (3-23-65), residual heat removal system torus cooling valves MO-3-10-39A and MO-3-10-34B, reactor water cleanup system return-to-reactor valve MO-3-12-68, 'A' ADS backup nitrogen solenoid valve SV-9130A, core spray system injection valve MO-3-14-11B, and feedwater check valve 3-6-28A.

The EIIS codes for the affected systems are as follows: Instrument Nitrogen - LK, Oxygen Analyzer - IK, High Pressure Coolant Injection - BJ, Residual Heat Removal - BO, Reactor Water Cleanup - CE, Core Spray - BM, and Feedwater - SJ.

Consequences of the Event:

For each valve listed, there is a redundant series isolation valve. The local leak rate test results of the redundant isolation valves were acceptable; therefore, containment integrity would have been maintained in the event of an accident.

Cause of the Event:

The cause of the excessive valve leakages will be investigated when the valves are disassembled and inspected. The results of these inspections will be submitted in a follow-up LER report within 30 days after the end of the current refueling outage.

Corrective Actions:

Unit 3 is presently in a refueling outage. All valves with unacceptable leak rates will be repaired and retested prior to returning Unit 3 to operation. These corrective actions will ensure that the total combined leakage rate for all Type B and C tests will be below 0.6 La.

Previous Similar Occurrences:

Peach Bottom LER's 2-85-01 and 3-85-05.

NRC Form 366B
(9-83)

LICENSEE EVENT REPORT (LER) FAILURE CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Peach Bottom Atomic Power Station - Unit 3	0 5 0 0 0 2 7 8	8 5	— 0 1 5	— 0 0	0 4 OF 0 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	B O	I S V	W 0 3 0	Y							
X	C E	I S V	W 0 3 0	Y							
X	L K	I S V	T 0 2 0	Y							
X	B M	I S V	W 0 3 0	Y							
X	S J	I S V	A 5 8 5	Y							

NRC FORM 366B
(9-83)

PHILADELPHIA ELECTRIC COMPANY

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October 29, 1985

Docket No. 50-278

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Peach Bottom Atomic Power Station - Unit 3

This LER addresses exceeding the total leakage rate limit for Type B and C tests.

Reference:	Docket No. 50-278
Report Number:	3-85-15
Revision Number:	00
Event Date:	September 30, 1985
Report Date:	October 29, 1985
Facility:	Peach Bottom Atomic Power Station RD 1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i).

Very truly yours,

W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
T. P. Johnson, NRC Resident Inspector

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