

NOTICE OF VIOLATION

Detroit Edison Company

Docket No. 50-341

As a result of the inspection conducted on February 6, 1988 to March 31, 1988 and in accordance with the "General Policy and Procedures for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1988), the following violations were identified:

1. Technical Specification 6.2.1 requires that the offsite organization for unit management and technical support shall be as show' on Figure 6.2.1-1. Technical Specification 6.2.2 requires that the unit organization shall be as shown on Figure 6.2.2-1.

Contrary to above, the licensee's current organization does not conform to Figures 6.2.1-1 and 6.2.2-1 in that:

- The Independent Safety Engineering Group reports to the Nuclear Safety Review Group Chairman vice the Director Nuclear Quality Assurance.
- The Director Nuclear Training replaced the positions of Director Nuclear Training-General and Director Operator Training.
- The General Supervisor Nuclear Fuel reports directly to the Vice President Nuclear Engineering and Services vice the Director of Nuclear Engineering.
- The Director of Nuclear Security position exists under the Plant Manager vice Superintendent of Services.
- The Nuclear Quality Assurance organization has been reorganized.
- The Assistant Operations Engineer has been eliminated from the unit organization.

This is a Severity Level V violation (Supplement I).

2. 10 CFR 50, Appendix B, Criterion VIII, "Identification and Control of Materials, Parts, and Components," requires that measures shall be established for the identification and control of materials, parts, and components, including partially fabricated assemblies. These measures shall assure that identification of the item is maintained by heat number, part number, serial number, or other appropriate means, either on the item or on records traceable to the item, as required throughout fabrication, erection, installation, and use of the item. These identification and control measures shall be designed to prevent the use of incorrect or defective material, parts, and components.

Contrary to the above, the measures established for control of parts did not assure against the use of incorrect parts in that a diesel fire pump alternator was replaced with an alternator of different make and number without being evaluated for suitability.

This is a Severity Level IV violation (Supplement I).

3. 10 CFR 50, Appendix B, Criterion XI, "Test Control," requires that test procedures shall include provisions for assuring that adequate instrumentation is available and used. UFSAR Section 17.2.11, "Test Control," states that the responsible supervisor makes certain that test equipment has the proper accuracy and is properly calibrated.

Contrary to the above, Paragraph 3.4 of Procedure POM 43.401.300 does not specify a calibrated timer as required equipment for conducting water leakage tests. Additionally, on March 17, 1988, the licensee was observed performing a water leakage test of Valve E11-F001B without a calibrated timer.

This is a Severity Level V violation (Supplement I).

4. Technical Specification 6.8.1.a. requires the applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978 be established, implemented and maintained.

Regulatory Guide 1.33, Revision 2, February 1978 Appendix A.1.c requires administrative procedures for equipment control.

Procedure POM 12.000.080, Conduct of Electrical Field Activities, Paragraph 7.1 states in part, "An "Interim Alteration Checklist" should be used for documenting the determination and retermination of electrical connections as well as other interim alterations that become necessary as part of testing, investigation, repair or replacement . . . ."

Procedure POM 12.000.080, Conduct of Electrical Field Activities, Paragraph 7.5.1 states "An independent second check of restoring to normal shall be performed for all interim alterations performed under work orders designated as Safety-Related on the Work Order Package Attachment A."

Contrary to the above, I&C personnel did not properly adhere to Administrative Procedure 12.000.080 in that a calibration check on safety related recorder B21R623A involving two lifted wires was done without the use of an interim alteration checklist and without independent verification.

This is a Severity Level IV Violation (Supplement I).

5. 10 CFR 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings," states in part, "Activities affecting quality shall be prescribed by documented . . . procedures . . . and shall be accomplished in accordance with these . . . procedures."

Procedure 24.000.02, "Shiftly, Daily, Weekly and Situational Required Surveillances," Attachment 10, specified the core spray system differential pressure be determined locally every 12 hours.

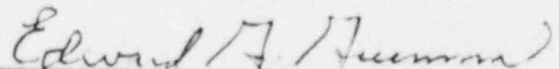
Contrary to the above, on nine separate occasions, between February 7 and 18, 1988, the differential pressure was determined in excess of 12 hours.

This is a Severity Level IV Violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, you are required to submit to this office within thirty days of the date of this Notice a written statement or explanation in reply, including for each violation: (1) corrective action taken and the results achieved; (2) corrective action to be taken to avoid further violations; and (3) the date when full compliance will be achieved. Consideration may be given to extending your response time for good cause shown.

MAY 27 1988

Dated \_\_\_\_\_

  
Edward G. Greenman, Director  
Division of Reactor Projects