

APPENDIX

U.S. NUCLEAR REGULATORY COMMISSION  
REGION IV

NRC Inspection Report: 50-313/86-11  
50-368/86-11

Licenses: DPR-51  
NPF-6

Dockets: 50-313  
50-368

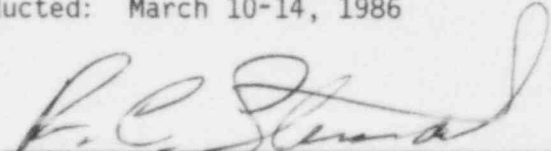
Licensee: Arkansas Power & Light Company  
P. O. Box 551  
Little Rock, Arkansas 72203

Facility Name: Arkansas Nuclear One (ANO), Units 1 and 2

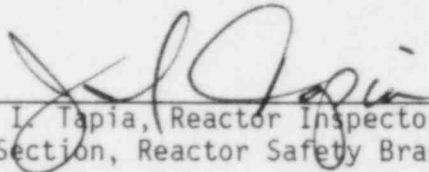
Inspection At: ANO site, Russellville, Arkansas

Inspection Conducted: March 10-14, 1986

Inspectors:

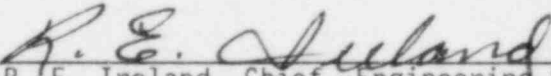
  
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R. C. Stewart, Reactor Inspector, Operations  
Section, Reactor Safety Branch

5/2/86  
Date

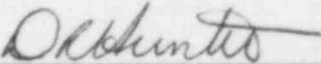
  
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J. I. Tapia, Reactor Inspector, Engineering  
Section, Reactor Safety Branch

5/2/86

Approved:

  
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R. E. Ireland, Chief, Engineering Section,  
Reactor Safety Branch

5/2/86  
Date

  
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D. R. Hunter, Chief, Reactor Projects  
Section B, Reactor Projects Branch

5/2/86  
Date

Inspection Summary

Inspection Conducted March 10-14, 1986 (Report 50-313/86-11)

Areas Inspected: Routine, unannounced inspection of licensee's design control program and facility modifications - condensate storage tank. In addition, the inspectors reviewed the Unit 1, final ILRT test report.

Results: Within the three areas inspected, no violations or deviations were identified.

Inspection Conducted March 10-14, 1986 (50-368/86-11)

Areas Inspected: Routine, unannounced inspection of licensee's design control program and facility modifications - condensate storage tank.

Results: Within the two areas inspected, no violations or deviations were identified.

DETAILS

1. Persons Contacted

Arkansas Power and Light Company (AP&L)

- \*J. Levine, Site Director
- \*D. Lomax, Plant Licensing Supervisor
- \*E. Ewing, Engineering & Technical Support
- \*C. Bean, QA Inspector, Plant Modifications
- \*B. Bata, QA Inspector
- \*L. Schempp, QC Manager
- \*H. Jones, Plant Modifications Manager
- \*P. Campbell, Plant Licensing Engineer
- D. Payne, Engineer
- G. Parks, Senior QC Inspector

The NRC inspector also contacted other site personnel including clerical and engineering.

\*Denotes those present at the exit interview conducted March 14, 1986.

2. Status of Previously Identified Items

(Closed) Unresolved Item (50-313/8434-01):

- a. Information was not available to demonstrate that volumetric examination of nozzle-to-vessel welds on the Unit 1 reactor vessel had been completed.

During this inspection, the NRC inspector reviewed the licensee's action tracking response data package dated December 3, 1984, containing sketches and information that reflect that the required 100 percent weld volumes were examined. This matter is resolved.

- b. Data Report Form NIS-1, "Owner's Data Report for Inservice Inspections," dated September 20, 1983, indicated only five of the eight vessel nozzles had been listed on the report.

During this inspection, the licensee provided the NRC inspector with a copy of the amended form dated December 11, 1984, which certifies inspection results of all eight nozzles examined. This matter is resolved.

- c. Data showed calibration sheets for several transducers; however, transducer identification numbers were not listed on data sheets for the nozzle examinations.

During this inspection, the NRC inspector was provided calibration data sheets showing all five transducer identification numbers being assigned to the single inspection head, B&W DB-12064, and as shown on B&W transducer beam spread measurement report. This matter is resolved.

- d. There was no documented evidence of either B&W or AP&L QA/QC overview of the ISI process.

During this inspection, the NRC inspector was provided a listing of eight surveillances conducted by AP&L QA during B&W ISI activities which meet AP&L surveillance procedural requirements. In discussing this matter, the licensee representatives acknowledged a past weakness in the operations surveillance activities relating to an overview of contractor activities. A proposed reorganization of the site QA/QC organizational structure has been approved and is currently being implemented. The new organization will incorporate an onsite general manager, nuclear quality. This matter is resolved.

3. AP&L Design Control Program

During this inspection, the NRC inspector conducted a review of the licensee's design change and modification control program to determine whether the licensee is implementing a QA program relating to the control of design changes and modifications that are in conformance with regulatory requirements, commitments in the application, and industry guides and standards.

a. Inspection Scope

The following quality assurance program manuals, procedures, and related documents were reviewed in conjunction with 10 CFR 50, Appendix B, Criterion III, "Design Control" and Criterion VI, "Document Control;" Regulatory Guide 1.64, Revision 2, June 1976; and ANSI Standard N45.2.11-1974:

- ° AP&L Quality Assurance Manual - Operations Topical Report, Revision 6, dated May 30, 1984.
- ° AP&L Design Control Procedure 1032.01, Revision 7, dated October 16, 1985.
- ° ANO-2 Technical Specification 4.8.1.1.2.
- ° AP&L Control of Station Modification Procedure 1000.13, Revision 10, dated September 25, 1985.
- ° Installation Technical Support Procedure 1000.10.
- ° Design Document Control Procedure 1032.11.

- Design Drawing Preparation, Review and Approval Procedure 1032.13.
- Design Change Approval, Form 1000.13D.
- Design Change Package Index, Form 1032.01F.
- Design Change Summary, Form 1032.01A.
- EQ Baseline Data, Form 1000.24F.
- Design Document Checklist, Form 1032.01B.
- Design Evaluation Questions, Attachment 3, Procedure 1032.01.
- Field Change Notices, Form 1032.01G.
- Control Room Procedure 1000.28, "Jumper & Lifted Lead Control"

In addition, the NRC inspector reviewed an AP&L QA audit entitled, "Operating Plant Surveillance Audit, Design Control," QAP-9-85, Revision 1, conducted during the period September 4 through December 13, 1985.

There were no significant findings identified in the letter of findings dated January 13, 1986.

b. Inspection Findings

The AP&L design changes and modifications program appears to provide the essential design change requirements as specified by regulatory requirements, commitments by the licensee, approved licensee procedures and industry guides and standards.

No violations or deviations were identified.

4. Facility Modifications - Condensate Storage Tank

The purpose of this inspection was to verify that the facility modification, which involves the addition of a new condensate storage tank (for Units 1 and 2), is completed in conformance with requirements in the facility license, Technical Specifications, 10 CFR, and applicable codes and standards. This condensate storage tank (CST) addition is being incorporated under design change packages (DCP) No. 82-2086 and constructed in three phases. The first phase involves only the civil portion and comprising the foundation, foundation piers, valve pits, pipe trenches and tornado missile protection walls.

### Inspection Scope

During this inspection, construction progress involved installation of the 42-inch drilled reinforced concrete piers (27) and portions of the pipe trenches.

The NRC inspector reviewed the DCP 82-2086 package in order to verify the following:

- a. That the design change and change notices were reviewed and approved in accordance with Technical Specifications and approved licensee procedures.
- b. That the licensee conducted a review and evaluation within previously established acceptance criteria.
- c. The design and construction conforms to industry codes and standards to which the facility was constructed.
- d. Licensee's QA/QC involvement in the design review process.

In addition, the NRC inspector reviewed the AP&L QA/QC program being implemented during construction, including AP&L Technical Specification APL-C-2401, "Technical Specification for Material Testing and/or Inspection Services," QC Instruction Nos. ANO/BF-1.00 and ANO/C-1.00.

In conjunction with the above, the NRC inspector examined concrete material test reports performed by Anderson Engineering & Test Company in addition to AP&L QC inspection reports performed by the AP&L onsite construction inspection staff.

During the revision and observation of work activities, the NRC inspector observed that eight NCRs and two deviations were written against a number of reinforced concrete piers during their installation. The resolution of those matters will be reviewed during a follow on inspection.

No violations or deviations were identified.

### 5. Review of Final ILRT Test Report (Unit 1)

The NRC inspector reviewed the final ILRT report submitted pursuant to the requirements of 10 CFR Part 50, Appendix J. The submitted report was found to conform to the data obtained by the NRC inspector during the conduct of the test. The review of the raw data is documented in NRC Inspection Report No. 50-313/84-36. The review conducted during this inspection period confirms the acceptability of the report submitted by the licensee.

6. Exit Interview

An exit interview was conducted March 14, 1986, with the personnel denoted in paragraph 1 of this report. The NRC resident inspector also attended this meeting. At this meeting the scope of the inspection and the findings were summarized.