

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

APRIL, 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

April 1 - 30

April 1, 0001 Hours. The Unit entered April with the reactor subcritical and Unit Off-Line in cold shutdown for first refueling outage.

April 30, 2400 Hours. Reactor in cold shutdown for first refueling outage.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

AMENDMENT 37 - Vent and Purge Isolation Valves Upgrade
Revision.

AMENDMENT 38 - Chlorine Detector Removal.

2. Facility or procedure changes requiring NRC approval.

There were no Facility or Procedure Changes Requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L22535	RHR Discharge Pressure Tech. Spec. Requirements Inst. 1E12-N022A/B/C		Conflict between instrument accuracy and Tech. Spec. Requirements.	Installed new 1E12-N512 A/B/C instruments.
L31439	LPCS Testable Check Bypass Valve 1E21-F333	M-1-1-84-003	No Potential Effect.	Remove Valve Per ECN-PFL-49-LS/ED-85.
L31440	HPCS Testable Check Bypass Valve 1E22-F354	M-1-1-84-003	No Potential Effect.	Remove Valve Per ECN-PFL-49-LS/ED-85.
L31441	RCIC Testable Check Bypass Valve 1E51-F355	M-1-1-84-003	No Potential Effect.	Remove Valve Per ECN-PFL-49-LS/ED-85.
L31442	LPCI INBD Testable Check Bypass Valve 1E12-F327A.	M-1-1-84-003	No Potential Effect.	Remove Valve Per ECN-PFL-49-LS/ED-85.
L31443	LPCI Outbd Testable Check Bypass Valve 1E51-F354	M-1-1-84-003	No Potential Effect.	Remove Valve Per ECN-PFL-49-LS/ED-85.
L40863	Barton Diff. Press Switches 1B21-N026.	M-1-1-84-106	Environmental Qualifications.	Replaced Per ECN-ED-141/PFL-158.
L40974	Barton Diff. Press. Switched 1B21-N044	M-1-1-84-106	Environmental Qualifications.	Replaced Per ECN-ED-141/PFL-158.
L43683	RCIC Isolation Valves For Level Switches 1E51-F331.	Leakage Through the packing.	Valve Leaking, Hampering Level Switch Calibration.	Cleaned Seating Surface.
L51383	1A RHR Min. Flow Valve 1E12-F064A.	Leakage through Packing	LTS-900-12 LLRT	Repacked Valve.

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WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L53596	S/D Cooling Testable check valve 1E12-F050B	Limit Switches incorrectly set.	Valve indicates closed when valve is actually open.	Upset limit switches.
L54558	H ₂ Recombiner Inlet Valve from U2 1HG002B	Limit Switches incorrectly set.	Valve opens, but will not full close from control room.	Reset limit switches per LES-EQ-112.
L55159	Limitorque Valve 1E51-F064.	Broken Clutch gear.	Motor Runs but does not engage with valve.	Replaced clutch gear.
L55957	RBCCW D/W outbd Isol. Valve 1WR040.	Bad Pinion Gear on Limit Switch.	LTS-100-30, LLRT.	Replaced Pinion gear on Limit Switch.
L55993	Mechanical Snubbers RH53-1562S.	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.
L56167	Mechanical Snubbers LP02-1059S	Bad Snubber	Failed LTS-500-4	Replaced Snubber.
L56232	Mechanical Snubber RH13-1154S.	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.
L56235	RCIC Pump Suction Valve 1E51-F010	Bad Main & Aux. Contactors	Valve Spuradically opens from Control Room.	Replaced Main and Aux. Contactors.
L56387	Mechanical Snubber LC01-1005S.	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L56388	Mechanical Snubber NG11-1003S.	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L56389	Mechanical Snubber RH-1213 H02S.	Bad Snubber.	Failed LTS-500-14	Replaced Snubber.

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CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L56423	Mechanical Snubber RH-1207-H09S	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L56424	Mechanical Snubber RH23-H07S	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L56425	Mechanical Snubbers RI109-1005S/1008S/ 1026S.	Bad Snubbers.	Failed LTS-500-14.	Replaced Snubbers.
L56426	Mechanical Snubber RR69-H09S.	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L56614	RHR Hx Vent Valve 1E12-F073A.	Pinch wire on Limit Switch	When Valve is full open No indication.	Repaired pinched wire.
L56902	RHR Test Return Valve 1E12-F021.	M-1-1-84-090	No Potential Effect.	Reset thermal O/L and Magnetic settings per ECN-ED-167-3.
L56952	RBCCW D/W outed Isol. Valve 1WR040.	Bad Packing.	Amperage Higher than acceptable criteria.	Repacked valve.
L57118	LPRM String 16-41.	Bad LPRM String 16-41.	Inoperble LPRM String	Replaced LPRM String 16-41.
L57140	RHR S/D cooling Isol. Valve 1E12-F009.	Misadjusted limit switch in Open Circuit.	Valve would not open with Diff. Press Across valve.	Readjusted limit switch.
L57162	RBCCW 1WR179 Valve MCC 136Y-2-Compt. A4 O/L Trip.	Bad Thermal O/L.	Thermal O/L Does Not Trip Manually during LES-GM-109.	Replaced Thermal O/L.

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SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L57163	RBCCW 1WR180 Valve Forward and Reverse contactors.	Sticking and Dirty contactor.	Forward and reverse contactors. Stick when manually actuated.	Cleaned and lubricated contactors.
L57175	H ₂ Recombiner Supply Valve from U2. 1HG018.	Burned Transformer in breaker.	Valve will not cycle open electrically.	Replace ^d burned transformer.
L57216	1AP71E, MCC 135X-1 Compt.-E3 for 1B21-F067C Valve.	Interconnecting Linkage was binding.	Contactor sticks when manually actuated.	Cleaned, Lubricated and realigned linkage.
L57233	1AP76E, MCC 135Y-2 Compt. B6 For 1E12-F006A Valve.	Bakelite on Main circuit Breaker is broken.	No Potential Effect, Valve cycled.	Replaced Breaker.
L57246	Mechanical Snubber RH40-1539S.	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L57247	Mechanical Snubber RH53-1565S and RH14-1047S.	Bad Snubbers.	Failed LTS-500-14.	Replaced Snubbers.
L57250	Mechanical Snubbers RH40-1548S/1544S/ 1554S.	Bad Snubbes.	Failed LTS-500-14.	Replaced Snubbers.
L57251	Mechanical Snubbers RR00-1004S.	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L57268	LPRM 16-41	LPRM Housing is Bent.	LPRM is obstructed from seating in Guide Tube.	Straighten and Grounded LPRM.
L57272	Mechanical Snubber RH-1209-H03S.	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.

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WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L57274	Mechanical Snubber RR17-1005S.	Bad Snubber	Failed LTS-500-14	Replaced Snubber.
L57275	Mechanical Snubber RR17-1008S.	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.
L57276	Mechanical Snubber RH50-1017S.	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.
L57277	Mechanical Snubber RR01-1032S.	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.
L57279	Mechanical Snubber RRC0-1011S	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.
L57280	Mechanical Snubber RHB4-1008S/1011S.	Bad Snubbers.	Failed LTS-500-14.	Replaced Snubbers.
L57281	Mechanical Snubber RR17-1003S/1004S/ 1007S	Bad Snubbers	Failed LTS-500-14.	Replaced Snubbers.
L57282	Mechanical Snubber RHB4-1007S and RH52-H09S.	Bad Snubbers.	Failed LTS-500-14.	Replaced Snubbers.
L57283	Mechanical Snubbers M-1302-28-84/22-110/ 21-40.	Bad Snubbers	Failed LTS-500-14.	Replaced Snubbers.
L57284	Mechanical Snubbers RR00-1030S/1061S.	Bad Snubbers	Failed LTS-500-14	Replaced Snubbers.

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WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L57285	Mechanical Snubbers RR00-1022S/1043S.	Bad Snubbers	Failed LTS-500-14.	Replaced Snubbers.
L57315	Mechanical Snubbers RR00-1062S.	Bad Snubbers.	Failed LTS-500-14.	Replaced Snubbers.
L57316	Mechanical Snubbers RR00-1014S.	Bad Snubbers.	Failed LTS-500-14.	Replaced Snubbers.
L57368	Mechanical Snubbers SC02-1045T/G.	Bad Snubber.	Failed LTS-500-14.	Replaced Snubbers.
L57369	Mechanical Snubber ER00-1059S.	Bad Snubber	Failed LTS-500-14.	Replaced Snubber.
L57465	LPCS Full Flow Test Valve 1E21-F012.	Motor Pinion Gear Key Sheared.	Inoperable Valve.	Replaced Pinion Gear Key.
L57480	Mechanical Snubber M-1302-30-52.	Bad Snubber.	Failed LTS-500-14.	Replaced Snubber.
L57483	"A" RHR LPCI Testable Check Valve 1R12-F041A set.	Limit Switches incorrectly set.	Valve has dual indication in control room, and will not go full closed.	Reset Limit Switch Cams.
L57517	Mechanical Snubbers M1302-26-110.	Bad Snubber.	Failed Stroke Check.	Replaced Snubber.
L57649	H ₂ Recombiner Valve From U2 1HG005B.	Bad Worm Gear Assembly and Tripper Finger Assembly.	Motor will not stop running on limitorque.	Replaced worm gear Assembly and Tripper Finger Assembly.

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SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L57695	A RHR Hx Vent Motor Operated Valve 1E12-F073A.	Cut and Pinched wires.	Motor keeps blowing control power fuses.	Repaired and relugged wires.
L57838	"A" VE Return Fan Inlet Damper 0VE06YA.	Actuator was bound at 90% open when deenergized.	Damper was stuck open.	Replaced Actuator.
L57873	1C71-K14A/E contacts.	Aux. Contact Clip had fallen off.	No control Room Alarm.	Replaced Clip.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, April 1 through April 30, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
86-011-00	03/17/86	Primary Containment Isolation Due to Operator ordering jumpers removed.
86-012-00	04/02/86	"A" VC Emergency Make-Up Train spurious trip of "A" Hi Radiation Monitor Due to calibration procedure.
86-013-00	04/03/86	Loss of Bus 141Y and Auto Start of "0" Diesel Generator.
86-014-00	04/12/86	Spurious Trip of NH ₃ Detector Due to a Broken Chem-Cassette Tape.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE 5/10/86
 COMPLETED BY James P. Peters
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: April 1986 GROSS HOURS IN REPORTING PERIOD: 719
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	12039.00
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	1642.00
7. HOURS GENERATOR ON LINE	0.0	0.0	11642.00
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9. GROSS THERMAL ENERGY GENERATED (MWH)	0.0	0.0	32213650
10. GROSS ELEC. ENERGY GENERATED (MWH)	0.0	0.0	10499394
11. NET ELEC. ENERGY GENERATED (MWH)	-10967	-36437	9968020
12. REACTOR SERVICE FACTOR	0.0%	0.0%	58.9%
13. REACTOR AVAILABILITY FACTOR	0.0%	0.0%	66.9%
14. UNIT SERVICE FACTOR	0.0%	0.0%	56.9%
15. UNIT AVAILABILITY FACTOR	0.0%	0.0%	59.0%
16. UNIT CAPACITY FACTOR (USING MDC)	-1.5%	-1.2%	47.0%
17. UNIT CAPACITY FACTOR(USING DESIGN MWe)	-1.4%	-1.2%	45.2%
18. UNIT FORCED OUTAGE RATE	0.0%	0.0%	17.4%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
 The First Refueling, Maintenance, Surveillance and Modification Outage began October 18, 1985 and will last into June, 1986.20.
 IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: June 18, 1986

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: 5/10/86
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: APRIL 1986

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	-14	17. _____	-15
2. _____	-15	18. _____	-15
3. _____	-15	19. _____	-16
4. _____	-15	20. _____	-15
5. _____	-15	21. _____	-15
6. _____	-15	22. _____	-16
7. _____	-15	23. _____	-16
8. _____	-15	24. _____	-16
9. _____	-15	25. _____	-16
10. _____	-16	26. _____	-15
11. _____	-15	27. _____	-15
12. _____	-15	28. _____	-16
13. _____	-15	29. _____	-17
14. _____	-15	30. _____	-16
15. _____	-15	31. _____	N/A
16. _____	-15		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373

UNIT NAME LaSalle One

DATE 5/10/86

COMPLETED BY James P. Peters

TELEPHONE (815)357-6761

REPORT MONTH APRIL 1986

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER		CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED					

There were no Unit shutdowns or power reductions during this reporting period

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-653-86	1E12-F009	Reset Limit Switch.
1-667-86	1E12-F009	Change Breaker.
1-668-86	1E12-F049A	Change Motor
1-677-86	1E12-N010A	Replace 3 Valve Manifold.
1-680-86	1E51-F063	Install Redundant Fault Protection.
1-681-86	1E51-F076	Install Redundant Fault Protection.
1-687-86	RHR S/D Cooling	LES-PC-06.
1-688-86	1E51-F076	Install Redundant Fault Protection.
1-690-86	1E12-N010CA	Repair 3 valve manifold.
1-697-86	1E12-F074B	Replace Breaker.
1-698-86	1E12-F073B	Replace Breaker.
1-700-86	1E32-F001E	Replace Motor
1-710-86	1E21-F012	Repair Limitorque.
1-717-86	1E22-F010	Re-Lug Limitorque.
1-718-86	HPCS	Snubbers
1-723-86	1E12-F063C	Fill Suppression Pool
1-727-86	1E12-F050A	Repack Valve
1-737-86	1E1?-F041A	Reset Limits.
1-740-86	1HG006B	U1/U2 Separation
1-748-86	1E21-F012	Repair Pinion Gear
1-754-86	1E12-F040B	Set Magnetics
1-756-86	1E12-F090A	Snubbers
1-759-86	A RHR WS System	Inoperable to incomplete modification.
1-761-86	LPCS Pump	LES-LP-101
1-762-86	LPCS Waterleg Pump	Inspect/Repair
1-777-86	A RHR D/W Spray	Install ILRT Spool Piece.
1-773-86	A RHR S/D Cooling LPCI Inj.	LES-RH-100
1-778-86	1B Fuel Pool Make-Up Pp.	Rewind Motor.
1-780-86	1E12-F073A	Inspect Blown Fuses
1-801-86	1E51-F064	Repack Valve
1-807-86	A RHR	Keep System Filled with Waterleg Pp OOS.
1-814-86	1E12-F027A	Install Limiter Plate.
1-815-86	1B HPCS DG	Change Oil.

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

APRIL 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

April 1-30

- April 1, 0001 Hours. The Unit entered April with the Reactor critical at 98% power and Generator on line at 1097 MWe.
- April 4, 0130 Hours. Reactor Power Reduced to 92% (1037 MWe) Due to CRD Exercising per LOS-AA-W1.
- April 4, 0700 Hours. Reactor Power increased to 95% (1078 MWe and 3208 Mwt).
- April 11, 1910 Hours. Reactor Power Reduced to 88% (995 MWe) Due to CRD Exercising Per LOS-AA-W1.
- April 12, 0700 Hours. Reactor Power Increased to 95% (1064 MWe and 3170 Mwt)
- April 12, 2200 Hours. Reactor Power Reduced to 51% (600 MWe) Due to control rod manipulations.
- April 16, 2300 Hours. Reactor Power Increased to Full Power, 99% (1103 MWe). Also, Power is drifting down due to Xenon Build In and Control Rods Held For Jet Pump Calibration.
- April 19, 0050 Hours. Reactor Power Reduced to 90% (1019 MWe) Due to control rod manipulation.
- April 19, 0700 Hours. Reactor Power Increased to 95% (1070 MWe) limited by flow control line.
- April 20, 0500 Hours. Reactor Power Reduced to 77% (861 MWe) Due to control rod manipulation.
- April 21, 1500 Hours. Reactor Power Increased to 97% (1093 MWe and 3259 Mwt) Limited by Condensate Polisher Flow.
- April 22, 2300 Hours. Reactor Power Decreased and Steady at 99% (1105 MWe and 3302 Mwt).
- April 25, 0245 Hours. Reactor Power Reduced to 92% (1034 MWe) Due Control Rod Manipulation.
- April 25, 1500 Hours. Reactor Power Increased and Steady at 96% (1080 MWe and 3238 Mwt).
- April 26, 0240 Hours. Reactor Power Reduced to 73% (850 MWe) for MSIV/TSV surveillances and EGC Test.
- April 26, 083 Hours. Reactor Power Ramp Held at 78% (888 MWe) Due to failure of A RR M/A Station.
- April 27, 0100 Hours. Reactor Power Increased to 92% (1032 MWe).
- April 27, 0315 Hours. Reactor Power Decreased to 80% (900 MWe) Per Load Dispatcher.

April 1-30

- April 28, 0700 Hours. Reactor Power Increased to 99% (1105 MWe and 3294 MWt) but drifting down with Xenon built in.
- April 30, 1500 Hours. Reactor Power Deceased to 96% (1082 MWe).
- April 30, 1530 Hours. Reactor Power Reduced to 75% (850 MWe) Due to repairs on Heater Drain Pump Forward Valve.
- April 30, 2400 Hours. Reactor Power Steady at 75% (850 MWe and 3272 MWt).

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

AMENDMENT 20 - Chlorine Detector Removal.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure change requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L56655	2B DG Fuel Oil Transfer Pump System.	Level Indicator 2CI-D0012 Slightly Out of Tolerance.	Failure of LOS-DG-M3	Recaled Level Switches.
L57135	HCU 06-15 2C11- D001-111 Valve.	Excessive Leakage	HUC 06-15 Requires Recharging once Per Day.	Replaced Cartridge Valve.
L57267	RCIC Water Leg Pump 2E51-C003.	Pump Impeller Clearance to High.	Pump Generates Excessive Dis- charge Pressure.	Readjust Pump Impeller Clearance.
L57337	H ₂ Recombiner Inlet Valve 2HG-FV-1	Improper Finger Base Assembly.	Valve Would Not Open During Performance of LOS-HG-Q1.	Replaced Finger Base
L57602	Div.-II Post Loca Low Bottle Pressure Alarm.	Pressure Switch 2PS- CM102 was out of Tolerance.	Alarm is Up with no actual Low Bottle Pressure.	Recalibrated Pressure Switch 2PS-CM102.
L57636	Mechanical Snubber RIA3-2990S.	Bad Snubber	Failed LTS-500-4.	Replaced Snubber.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, April 1 through April 30, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
86-006-00	03/02/86	Reactor Water Clean-Up Isolation Due to Mispositioned Valves from Procedural Problem.
86-007-00	04/04/86	Excessive Static-O-Ring Pressure Switch Drift High Out of Tolerance.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE 5/10/86
 COMPLETED BY James P. Peters
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: April, 1986 GROSS HOURS IN REPORTING PERIOD: 719
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>719</u>	<u>2775.59</u>	<u>8164.99</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>29.83</u>	<u>29.83</u>
7. HOURS GENERATOR ON LINE	<u>719</u>	<u>2748.44</u>	<u>7984.74</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>2218512</u>	<u>8171616</u>	<u>23680168</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>740608</u>	<u>2731931</u>	<u>7842000</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>717755</u>	<u>2651337</u>	<u>7474552</u>
12. REACTOR SERVICE FACTOR	<u>100%</u>	<u>96.4%</u>	<u>60.7%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100%</u>	<u>97.4%</u>	<u>60.9%</u>
14. UNIT SERVICE FACTOR	<u>100%</u>	<u>95.4%</u>	<u>59.4%</u>
15. UNIT AVAILABILITY FACTOR	<u>100%</u>	<u>95.4%</u>	<u>59.4%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>96.3%</u>	<u>88.8%</u>	<u>53.7%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>92.6%</u>	<u>85.4%</u>	<u>51.6%</u>
18. UNIT FORCED OUTAGE RATE	<u>0.0%</u>	<u>4.5%</u>	<u>20.9%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): The Units First Refuel is scheduled to start September 1, 1986.			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP			<u>N/A</u>

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: 5/10/86
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: April 1986

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	1059	17. _____	1046
2. _____	1053	18. _____	1035
3. _____	1050	19. _____	1024
4. _____	1042	20. _____	928
5. _____	1043	21. _____	1032
6. _____	1041	22. _____	1046
7. _____	1037	23. _____	1058
8. _____	1032	24. _____	1050
9. _____	1030	25. _____	1036
10. _____	1024	26. _____	921
11. _____	1010	27. _____	913
12. _____	1014	28. _____	1055
13. _____	653	29. _____	1043
14. _____	766	30. _____	974
15. _____	871	31. _____	N/A
16. _____	1021		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373

UNIT NAME LaSalle Two

DATE 5/10/86

COMPLETED BY James P. Peters

TELEPHONE (815)357-6761

REPORT MONTH APRIL 1986

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
18	4/12/86		S	8.33	H-Adjustment of core Flux.	5	Control Rod Manipulations

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES</u> <u>ACTUATED</u>	<u>NO & TYPE</u> <u>ACTUATIONS</u>	<u>PLANT</u> <u>CONDITION</u>	<u>DESCRIPTION</u> <u>OF EVENT</u>
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There were no safety relief valves operated for Unit #2 during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-162-86	RCIC	Waterleg Pump Testing.
2-166-86	RHR S/D Cooling Valves	LIS-RH-412
2-170-86	2B DG Cooling Water System	Modify Control Switch
2-175-86	RCIC	Repair Oil Relief and Change Oil
2-176-86	RCIC Steam Supply Drain Trap 2E51-D003	Repair Steam Leak.
2-177-86	2E51-N010	Repair Leak.
2-178-86	2DG08CB	Move Piping Interference For EMD.
2-179-86	U2 LPCS Pump	Lubrication.
2-180-86	2E51-F012	Troubleshooting.
2-181-86	2DG08CB	Replace Bolts.
2-187-86	RCIC Water Leg Pump	Change Orifice.
2-189-86	2DG061A	Re-Condition Valve.
2-192-86	2A DG Cooling Water System	Wiring Modification
2-196-86	2DG01P	Coupling Lubrication
2-197-86	2DG01K	Clean Lube Oil Separator Ejector Screen.
2-198-86	RHR S/D Cooling	LIS-RH-212.

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

May 8, 1986

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period April 1, 1986 through April 30, 1986.

Very truly yours,

for R. J. Diederich

G. J. Diederich
Station Manager
LaSalle County Station

GJD/JPP/jdp

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
L. J. Anastasia, PIP Coordinator SNED
J. E. Ellis, GE Resident
H. E. Bliss, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle

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