

MONTHLY OPERATING REPORT - FEBRUARY 1986

At the beginning of the report period, Oyster Creek was operating at approximately 650 MWe.

On February 2, Standby Gas Treatment System (SGTS) I was declared inoperable after routine testing identified a failed inlet temperature transmitter which prevented system heaters from energizing. Following repairs, the system was retested and declared operable on February 4.

On February 4, the reactor Cleanup System was removed from service to facilitate sealant injection repairs to valves V-16-22C and V-16-233. Power was reduced to 602 MWe prior to returning the Cleanup System to service to minimize the effects of a flux increase due to cold water addition. The Cleanup System was returned to service later that day and power was subsequently increased to approximately 650 MWe. Permanent repairs are planned for the next refueling outage.

On February 4, Core Spray System II was removed from service to repair two minor piping leaks. As a target of opportunity, associated booster pump breaker undervoltage devices were inspected. Following completion of repairs and inspections, the system was retested and declared operable on February 5.

On February 6, Trunnion Room temperature increased due to failure of recirculation fan RF 1-6. RF 1-7 was placed in service and temperatures returned to normal. Inspection of RF 1-6 revealed a defective drive belt. Power was decreased to approximately 495 MWe to reduce radiation exposure during belt replacement. RF 1-6 was returned to service and RF 1-7 secured. RF 1-7 is only used as backup, due to its degraded condition, and is scheduled for repair during the next refueling outage. Plant load was returned to 650 MWe on February 7.

On February 11, all power rods were essentially withdrawn and end of fuel cycle "coastdown" began.

On February 20, during a routine surveillance test, reactor low level sensors RE05A1 and RE05B1 setpoints were found to be out of specification. The sensors were subsequently adjusted and the test completed satisfactorily. The event required a one (1) hour notification to the NRC.

On February 21, a high conductivity alarm for "A" North condenser annunciated accompanied by increasing conductivity indication. "A" North section of the condenser was removed from service following a load reduction to approximately 475 MWe. A leaking tube (1) was identified on February 22 and plugged. The condenser was returned to service and load increased to 560 MWe later that day. Maximum plant load was achieved on February 23. Power stabilized at 612 MWe on February 25 after establishing Xenon equilibrium.

On February 27, reactor low level sensor RE05A1 failed surveillance testing and was determined to be inoperable. This action required entering a technical specification "LOO" action statement and necessary notifications were made. The sensor was subsequently replaced, calibrated, tested satisfactorily and returned to operable status the same day.

Plant load at the end of the report period was 602 MWe.

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Monthly Operating Report

The following Licensee Event Reports were submitted during the month of February 1986:

Licensee Event Report 50-219/85-026 - On December 18, 1985 a recirculation pump tripped and its discharge valve could not be immediately closed. Reverse flow occurred in the idle loop and was electronically added to the flow in the other four recirculation loops. This resulted in the flow biased Average Power Range Monitor (APRM) setpoints for the scram and rod block functions to be less conservative than those allowed by Technical Specifications.

Licensee Event Report 50-219/86-001 - On January 17, 1986, three out of four reactor low level scram sensors were found out of specification during the monthly surveillance. Upon discovery, the setpoints were immediately adjusted within acceptable ranges. On January 20, two of the sensors were retested and both were found in specification, but one failed to reset when valved back into service. The switch was determined to have failed and was replaced. During the additional testing of the failed sensor, its monthly out-of-service limit of 60 minutes was exceeded, and a shutdown was commenced. The shutdown was terminated at 0055 hours on January 21 when a new sensor was placed in service. The cause of the occurrence has been attributed to instrument drift, and the cause of the sensor failure is being investigated at the vendor's facilities.

Licensee Event Report 50-219/86-002 - On January 24, 1986 at 1430 hours, a Safety Related Hydraulic Snubber (NQ-2-S7) was disconnected from the Containment Spray piping rendering this snubber inoperable. This action placed the plant in the Limiting Condition for Operation (LCO). The snubber was disconnected during execution of a Short Form generated to exchange the front paddle on the snubber. The snubber was then reconnected to the pipe, making it operable. The removed paddle was used to repair a feedwater snubber. The cause of the event is attributed to personnel error when the Group Shift Supervisor (GSS) approved a maintenance short form which approved maintenance reordering a safety related snubber inoperable.

OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: FEBRUARY, 1986
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): COASTDOWN -
APPROXIMATELY 2 NET MWe PER DAY
11. REASON FOR RESTRICTION, IF ANY: NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	672.0	1416.0	141889.0
13. HOURS RX CRITICAL	672.0	1416.0	93863.4
14. RX RESERVE SHTDWN HRS	0.0	0.0	469.7
15. HRS GENERATOR ON-LINE	672.0	1416.0	91475.1
16. UT RESERVE SHTDWN HRS	0.0	0.0	755.8
17. GROSS THERM ENER (MWH)	1242000	2648000	151525469
18. GROSS ELEC ENER (MWH)	420780	899170	51189855
19. NET ELEC ENER (MWH)	405019	865876	49174477
20. UT SERVICE FACTOR	100.0	100.0	64.5
21. UT AVAIL FACTOR	100.0	100.0	65.0
22. UT CAP FACTOR (MDC NET)	97.2	98.6	55.9
23. UT CAP FACTOR (DER NET)	92.7	94.1	53.3
24. UT FORCED OUTAGE RATE	0.0	0.0	10.2
25. FORCED OUTAGE HRS	0.0	0.0	10435.6
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION): REFUELING, APRIL 12, 1986, 6 MONTHS			
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:		N/A	

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET #50-219
UNITOYSTER CREEK #1
REPORT DATEMARCH 4, 1986
COMPILED BYANTHONY V. SAVINO
TELEPHONE #609-971-4456

MONTH FEBRUARY, 1986

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	625	16.	607
2.	624	17.	603
3.	624	18.	602
4.	621	19.	599
5.	624	20.	597
6.	608	21.	575
7.	624	22.	485
8.	625	23.	591
9.	620	24.	595
10.	620	25.	587
11.	619	26.	586
12.	616	27.	584
13.	613	28.	583
14.	611		
15.	609		

REFUELING INFORMATION - February, 1986

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: April 12, 1986

Scheduled date for restart following refueling: October 12, 1986

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes

Scheduled date(s) for submitting proposed licensing action and supporting information:

April, 1986

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core	= 560
(b) in the spent fuel storage pool	= 1204
(c) in the dry storage	= 148
(d) in temporary storage	= 60

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present licensed capacity: 2600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Reracking of the fuel pool is in progress. Six (6) out of ten (10) racks have been installed to date. When reracking is completed, discharge capacity to the spent fuel pool will be available until 1990 refueling outage.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-219
UNIT NAME Oyster Creek
DATE March 1986
COMPLETED BY R. Baran
TELEPHONE 971-4640

REPORT MONTH February 1986

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
N/A		N/A	N/A	A	N/A	N/A	N/A	N/A	Feb. 21: Reduced plant load from 620 MWe to 473 MWe (23.7% load reduction) to repair a tube leak in "A" North condenser.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source



GPU Nuclear Corporation

Post Office Box 388
Route 9 South
Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

March 11, 1986

Director
Office of Management Information
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. George Busch at (609) 971-4643.

Very truly yours,


Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:KB:dsm(0170A)
Enclosures

cc: Director (10)
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Washington, DC 20555

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NRC Resident Inspector
Oyster Creek Nuclear Generating Station

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