

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) River Bend Station	DOCKET NUMBER (2) 0 5 0 0 0 4 5 8	PAGE (3) 1 OF 0 3
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TITLE (4)
Loss of Feedwater and Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)														
1	1	2	1	8	5	8	5	0	4	1	0	1	0	5	0	9	8	6			0	5	0	0	0
																					0	5	0	0	0

OPERATING MODE (9) 2	POWER LEVEL (10) 0 0 3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
		<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
		<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)						
		<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
		<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)							
		<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)							
		<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Dan Williamson, Operations Supervisor	TELEPHONE NUMBER 5 0 4 6 3 5 - 6 0 9 4
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 11/21/85 at 0122 with the unit in operational condition 2 (startup), a reactor scram occurred from 2.7 percent power on low reactor water level. The low level resulted from a trip of the reactor feedwater pump and inability to place a standby feedwater pump in service prior to the level decrease to the low level scram trip setpoint. Level was manually restored by Reactor Core Isolation Cooling at a minimum level of +2 inches.

There was no impact on the health and safety of the public.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	— 0 4 1	— 0 1	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 388A's) (17)

REPORTED CONDITION

On 11/21/85 at 0025 with the unit in operational condition 2 (startup), Reactor Feedwater Pump (RFP) B tripped for unknown causes. The operator restarted the pump and restored reactor water level to normal. In order to investigate the cause, RFP C was started with its discharge Motor Operated Valve (MOV 26C) left shut in the event RFP B tripped again. At 0117 RFP B tripped when its auxiliary oil pump was secured. The operator mistakenly believed that a normal oil supply was available. An attempt to open the discharge MOV for RFP C failed with the breaker tripping on overload. RFP B and its auxiliary oil pump was restarted, but an attempt to open its discharge valve also failed. At 0123 the reactor scrambled when level decreased to the low level scram setpoint. Reactor Core Isolation Cooling (RCIC) (IEEE:BN) was manually started to restore reactor level. The lowest level indicated on narrow range instrumentation was +2 inches (164 inches above Top of Active Fuel). At 0127 the RFP B discharge valve was opened and RCIC secured.

The cause of the initial RFP B trip was believed to have resulted from a main oil pump trip on overload due to low oil temperature. Investigation into the inability to open RFP B and C discharge valves at first determined that the probable cause was due to high differential pressure across the discharge valves. This would result when the feedwater to condenser recirculation valve (FV104) is open as it was in this case to aid in reactor level control. However, subsequent investigations into a similar incident (reported in LER

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

85-60) revealed that the RFP discharge motor operated valve torque switch settings were too low. These torque switch settings were reset to their proper values.

The operations staff also issued a memorandum reminding personnel of the operation of the RFP oil system. The system is to be addressed in the licensed operator requalification training which began on 1/3/86.

The health and safety of the public was not affected during this event.



GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775
AREA CODE 504 635-6094 346-8651

May 9, 1986
RBG- 23642
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

River Bend Station - Unit 1
Docket No. 50-459

Please find enclosed Licensee Event Report No. 85-041
Revision 1 for River Bend Station - Unit 1. This report is
submitted pursuant to 10CFR50.73.

Sincerely,

J. E. Booker
Manager-Engineering,
Nuclear Fuels & Licensing
River Bend Nuclear Group

WKS
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BEH
JEB/TFP/DRG/BEH/je

cc: U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

INPO Records Center
1100 Circle 75 Parkway
Atlanta, GA 30339-3064

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