

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNRB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) Catawba Nuclear Station	DOCKET NUMBER (2) 05000413	PAGE (3) 1 of 1
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TITLE (4)  
Unanalyzed Postulated Single Failure Affecting Steam Generator Tube Rupture Analysis

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)
11	18	97	97	- 009	- 00	12	18	97	Unit 2	05000414
OPERATING MODE (9) 1 POWER LEVEL (10) 100 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11) 20.402(b) _____ 20.405(c) _____ 50.73(a)(2)(iv) _____ 73.71(b) _____ 20.405(a)(1)(i) _____ 50.36(c)(1) _____ X 50.73(a)(2)(v) _____ 73.71(c) _____ 20.405(a)(1)(ii) _____ 50.36(c)(2) _____ 50.73(a)(2)(vii) _____ OTHER (Specify in _____ 20.405(a)(1)(iii) _____ X 50.73(a)(2)(i) _____ 50.73(a)(2)(viii)(A) _____ Abstract below and _____ 20.405(a)(1)(iv) _____ 50.73(a)(2)(ii) _____ 50.73(a)(2)(viii)(B) _____ in Text, NRC Form _____ 20.405(a)(1)(v) _____ 50.73(a)(2)(iii) _____ 50.73(a)(2)(x) _____ 366A) _____										

LICENSEE CONTACT FOR THIS LER (12)

NAME RL Bair, Safety Review Group Manager	TELEPHONE NUMBER AREA CODE (803) 831-3743
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS

SUPPLEMENTAL REPORT EXPECTED (14)

X YES (if yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			01	19	98

**ABSTRACT** (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)  
 During design review, a more limiting Steam Generator (S/G) Tube Rupture (SGTR) accident scenario than previously analyzed in Updated Final Safety Analysis Report (UFSAR) Chapter 15 was found. In the limiting failure for the scenario, there is a loss of offsite power with failure of one of the 125 VDC Vital I&C Distribution Centers, EDE or EDF, resulting in the loss of ability to isolate auxiliary feedwater flow to two S/Gs and loss of control power to two S/G power operated relief valves. Local manual action would be necessary to prevent S/G overflow. Due to its complexity, a complete evaluation of this event has not yet been completed.

The cause of this event has not yet been determined; however, this event has similarities to an event reported in Licensee Event Report (LER) 413/97-001. Conservative administrative controls on primary and secondary coolant specific activities have been put in place to ensure offsite doses are bounded by previous analyses. Further investigation to identify the root cause of this event and corrective actions is being conducted and will be reported in a supplement to this LER.