

CERTIFICATE OF COMPLIANCE
FOR RADIOACTIVE MATERIALS PACKAGES

U.S. NUCLEAR REGULATORY COMMISSION

1. a. CERTIFICATE NUMBER	b. REVISION NUMBER	c. PACKAGE IDENTIFICATION NUMBER	d. PAGE NUMBER	e. TOTAL NUMBER PAGES
6294	23	USA/6294/AF	1	3

2. PREAMBLE

- This certificate is issued to certify that the packaging and contents described in Item 5 below, meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

a. ISSUED TO (Name and Address)

ABB Combustion Engineering
Nuclear Power, Inc.
P.O. Box 500
Windsor, CT 06095-0500

b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION:

Combustion Engineering, Inc. application
dated July 27, 1990, as supplemented

c. DOCKET NUMBER

71-6294

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

(1) Model No.: UNC-2901

(2) Description

A maximum 10.80-inch square by 30-inch long inner container constructed of minimum 14-gauge steel, with bolted and gasketed top flange closure and sealed welded bottom sheet. Inner container is centered and supported in a 22.5-inch ID by 34-inch high 18-gauge steel drum with 16-gauge head and DOT Specification 17H closure by asbestos or ceramic sheet, plywood, hardboard, and insulating material. Gross weight of the package is 660 pounds.

(3) Drawings

The packaging is constructed in accordance with Combustion Engineering, Inc., Drawing Nos. D-5007-8086, Rev. 6, and B-5007-8112, Rev. 1.

(b) Contents

(1) Type and form of material

- Sintered uranium oxide pellets and rejected pellets enriched to a maximum 5.0 w/o in the U-235 isotope.
- Uranium oxide as powder enriched to a maximum 5.0 w/o in the U-235 isotope.
- U₃O₈ powder, placed in polyethylene bags then pressed and compacted into blocks, with a maximum enrichment of 4.5 w/o in the U-235 isotope. Water may be injected into the blocks.

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7. For the contents specified in 5(b)(1)(i), the pellet trays and wood spacers must provide a snug axial and cross sectional fit in the inner container. For packages with fewer than 16 loaded pellet trays, wood spacers or pellet trays with wood spacers inside must be substituted for pellet trays.
8. For the contents specified in 5(b)(1)(ii), powder cans and wood spacers must provide a snug axial and cross sectional fit in the inner container. For packages with fewer than two loaded powder cans, a wood spacer or a powder can with a wood spacer must be substituted for the powder can.
9. For the contents specified in 5(b)(1)(iii), the packaging may be constructed in accordance with Combustion Engineering, Inc., Drawing Nos. D-5018-8454, ^{(a) Rev.} 1 and D-5007-8112, Rev. 1.
10. In addition to the requirements of Subpart G of 10 CFR Part 71:
 - (i) Each packaging must meet the acceptance tests and be maintained in accordance with Chapter 8 of the application; and
 - (ii) The package must be prepared for shipment and operated in accordance with the Operating Procedures of Chapter 7 of the application, as supplemented.
11. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR §71.12.
12. Expiration date: March 31, 2001.

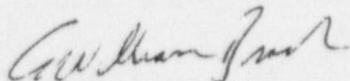
REFERENCES

Combustion Engineering, Inc. application dated July 27, 1990.

Supplements dated: October 19, 1990; January 27, and July 28, 1994; August 17, 1995; and July 14, 1998.

ABB Combustion Engineering Nuclear Power, Inc., supplement dated: June 10, 1999

FOR THE U.S. NUCLEAR REGULATORY COMMISSION



E. William Brach, Director
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards

Date: 6/13/99

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7. For the contents specified in 5(b)(1)(i), the pellet trays and wood spacers must provide a snug axial and cross sectional fit in the inner container. For packages with fewer than 16 loaded pellet trays, wood spacers or pellet trays with wood spacers inside must be substituted for pellet trays.
8. For the contents specified in 5(b)(1)(ii), powder cans and wood spacers must provide a snug axial and cross sectional fit in the inner container. For packages with fewer than two loaded powder cans, a wood spacer or a powder can with a wood spacer must be substituted for the powder can.
9. For the contents specified in 5(b)(1)(iii), the packaging may be constructed in accordance with Combustion Engineering, Inc., Drawing Nos. D-5018-8454, Rev. 1 and D-5007-8112, Rev. 1.
10. In addition to the requirements of Subpart G of 10 CFR Part 71:
 - (i) Each packaging must meet the acceptance tests and be maintained in accordance with Chapter 8 of the application and
 - (ii) The package must be prepared for shipment and operated in accordance with the Operating Procedures of Chapter 7 of the application, as supplemented.
11. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR §71.12.
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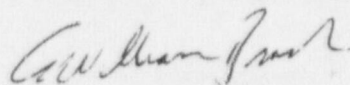
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FOR THE U.S. NUCLEAR REGULATORY COMMISSION



E. William Brach, Director
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards

Date: _____

6/18/99



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

APPROVAL RECORD
Model No. UNC-2901
Certificate of Compliance No. 6294
Revision No. 23

By letter dated June 10, 1999, ABE Combustion Engineering Nuclear Power, Inc. requested that the certificate holder of Certificate of Compliance No. 6294 for the Model No. UNC-2901 package be changed from Combustion Engineering, Inc., to ABB Combustion Engineering Nuclear Power, Inc. ABB Combustion Engineering Nuclear Power, Inc., has accepted responsibility for the completeness and accuracy of the statements and representations of the previous certificate holder. ABB Combustion Engineering Nuclear Power, Inc., will be responsible for maintenance of the certificate, the safety analysis report for the package designs, and the quality assurance records in accordance with 10 CFR §71.91(c). ABB Combustion Engineering Nuclear Power, Inc., has been issued Quality Assurance Program Approval for Radioactive Material Packages No. 0090, under Subpart H of 10 CFR Part 71.

The certificate has been revised to show ABB Combustion Engineering Nuclear Power, Inc., as certificate holder. These changes do not affect the ability of the packages to meet the requirements of 10 CFR Part 71.

E. William Brach, Director
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards

Date: 6/13/99

ATTACHMENT 7

CE-B1 Container

Docket Number COC 71-9272