

H. B. Barron Vice President, McGuire Nuclear Generation Department Duke Power Company
A Duke Energy Company

McGuire Nuclear Station MG01VP 12700 Hagers Ferry Rd. Huntersville, NC 28078-9340

(704) 875-4800 OFFICE (704) 875-4809 FAX

July 10, 1997

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

Subject:

McGuire Nuclear Station, Unit 1

Docket No. 50-369

Request for Alternative Examination, M97-002

Dear Sir:

Duke Power Company has determined that it is unable to completely satisfy the requirements for the augmented reactor vessel shell weld examination for McGuire Nuclear Station, Unit I as specified in 10CFR50.55a(g)(6)(ii)(A). Pursuant to provisions of 10CFR 50.55a(g)(6)(ii)(A)(5), please find enclosed information to support the determination and a proposed alternative examination that will provide and acceptable level of quality and safety.

Should questions arise regarding this matter, please contact J.M. Washam at (704) 875-4181.

Very truly yours,

HB Bour

H.B. Barron

9707230272 970710 PDR ADOCK 05000369 PDR IE101/1



U.S. Nuclear Regulatory Commission July 10, 1997 Page 2

xc. Mr. L.A. Reyes
Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

Mr. Victor Nerses, Project Manager Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission One White Flint North, Mail Stop 9H3 Washington, DC 20555

Mr. Scott M. Shaeffer, Senior NRC Resident Inspector, McGuire McGuire Nuclear Station

Duke Energy Corporation

Station: McGuire Nuclear Station, Unit 1

10 Year Interval Request For Alternative No. 97-002

Pursuant to 10 CFR 50.55a(g)(6)(ii)(A)(5), Duke Energy has determined that it is unable to conform with the examination requirements of 10 CFR 50.55a(g)(6)(ii)(A) for McGuire Nuclear Station, Unit 1. Information is therefore being submitted in support of this determination, and a request for alternative pursuant to 10 CFR 50.55a(g)(6)(ii)(A)(5) is being sought from the requirements of 10 CFR 50.55a(g)(6)(ii)(A).

Background:

In response to NRC Information Notice 96-32, "Implementation of 10 CFR 50.55a(g)(6)(ii)(A) "Augmented Examination of Reactor Vessel", Duke Energy has reviewed the information contained in this notice for applicability to its facilities and has taken action to avoid or mitigate the effects of limited examinations. These actions are taken to eliminate and/or reduce the concerns expressed in this Information Notice.

Because of concerns regarding the scope of examination of reactor vessels, the NRC issued, in 1992, 10 CFR 50.55a(g)(6)(ii)(A), "Augmented Examination of Reactor Vessel", which contains requirements for an augmented examination of reactor vessels. The rule requires the licensee to implement, before the time required by normal updating of the inservice inspection (ISI) program, provisions in the 1989 Edition of the ASME, Boiler and Pressure Vessel Code, Section XI, to examine "essentially 100%" of the length of all reactor vessel shell welds. "Essentially 100%" examination as used in 10 CFR 50.55a(g)(6)(ii)(A)(2) means "more than 90% coverage of the examination volume of each weld, where the reduction in coverage is due to interference by another component, or part geometry."

In many cases, licensees have determined that the overall average examination coverage for reactor vessel shell welds may be more than 90%. However, the corresponding examination coverage for individual welds may be substantially less than 90%. In these cases, licensees are unable to completely satisfy the requirements for the augmented reactor vessel examination. They must therefore propose an alternative that would provide an acceptable level of quality and safety.

The licensee must expend all efforts using the latest methods and techniques to achieve acceptably adequate examinations during weld inspections. When examinations cannot be completed with "essentially 100%" coverage, then a request for alternatives from the Code of Federal Regulations must be submitted.

I. System/Components For Which Alternative Is Requested:

In response to the requirements of 10 CFR 50.55a(g)(6)(ii)(A) relief is requested for seven McGuire Unit 1 reactor pressure vessel shell welds specified in Examination Category B-A Item B1.10. These welds, listed below, were examined and found to have 90% coverage or less. Attachment 1 provides examination data for these welds which were examined during Refueling Outage 7, and for which relief is requested from the requirement for augmented examination of the reactor vessel.

a. Reactor Vessel Shell Welds

Weld Numbers	Item Numbers
1RPV10-442	B01.011.003
1RPV1-442A	B01.012.001
1RPV1-442B	B01.012.002
1RPV1-442C	B01.012.003
1RPV3-442A	B01.012.007
1RPV3-442B	B01.012.008
1RPV3-442C	B01.012.009

II. 10CFR50 Requirement:

10 CFR 50.55a(g)(6)(ii)(A) requires that all licensees shall augment their reactor vessel examination requirements at least once for the reactor vessel shell welds specified in Item B1.10 of Examination Category B-A "Pressure Retaining Welds in Reactor Vessel" in Table IWB-2500-1 of Subsection IWB of the 1989 Edition of Section XI, Division 1, of the ASME Boiler and Pressure Vessel Code. 10 CFR 50.55a(g)(6)(ii)(A) further states that "for the purpose of this augmented examination essentially 100% as used in Table IWB-2500-1 means more than 90 % of the examination volume of each weld, where the reduction in coverage is due to interference by another component, or part geometry." The augmented examination, when not deferred in accordance with the provisions of 50.55a(g)(6)(ii)(A)(3), shall be performed in accordance with the related procedures specified in the Sect.on XI Edition and Addenda applicable to the inservice inspection interval in effect on September 8, 1992, and may be used as a substitute for the reactor vessel shell weld examination scheduled for implementation during the inservice inspection interval in effect on September 8, 1992. 10 CFR 50.55a further states that "Licensees that make a determination that they are unable to completely satisfy the requirements for the augmented reactor vessel shell weld examination specified in 10 CFR 50.55a(g)(6)(ii)(A) shall submit information to the Commission to support the determination and shall propose an alternative to the examination requirements that would provide an acceptable level of quality and safety."

III. Requirement for which Alternative is Requested:

An alternative is being requested to the requirement of obtaining "essentially 100%" examination volume of weld coverage for McGuire Unit 1 reactor vessel B1.10 welds listed in section I and shown in Attachment 1.

IV. Basis for requesting Alternative:

If licensees make a determination that they are unable to completely satisfy the requirements for the augmented reactor vessel shell weld examination specified in 10 CFR 50.55a(g)(6)(ii)(A); then 10 CFR 50.55a(g)(6)(ii)(A)(5) requires the licensee to submit information to the Commission to support this determination and to propose an alternative to the examination requirements that would provide an acceptable level of quality and safety. The licensee may use the proposed alternative when authorized by the Director of the Office of NRR.

Attachment 2 provides the calculations documenting the actual amount of Code required examination coverage obtained. A combination of multiple angles and ultrasonic techniques was used to obtain the maximum coverage possible. The use of an alternate transducer head provided increased coverage through optimum transducer arrangement for scanning close to obstructions. However, during the ultrasonic examination of the welds referenced below and listed in Attachment 1 of this alternative, the greater than 90% coverage required per 10 CFR 50.55a(g)(6)(ii)(A)(2) could not be obtained due to geometry and actual physical barriers.

Reactor Vessel Lower Shell to Bottom Head Weld (1RPV10-442) (Item Number B01.011.003):

This examination was limited to 59.00% aggregate coverage of the required weld volume. The principal limitation for this weld is six core guide lugs welded to the vessel ID just above the weld on the lower shell section, whose presence restricts the scanning surface in that area and limits the examination coverage. These lugs are approximately 19 inches tall, 27 inches wide and extend 8 inches radially toward the center of the vessel, the lugs are positioned 60 Degrees around the vessel circumference. These dimensions include the attachment weld radius which physically limited the contact transducer head travel during examination.

Reactor Vessel Upper Shell Long Seam Weld at 60 Degree(1RPV1-442A) (Item Number B01.012.001):

This examination was limited to 90.00% aggregate coverage of the required weld volume. The principal limitation for this weld involves nozzle interference which divides the weld into two parts. The Reactor Vessel Nozzle configuration prevents examination of the weld at the intersection of the weld and nozzle. The specific limitations are identified on drawing 1213926D.

Reactor Vessel Upper Shell Long Seam Weld at 180 Degree (1RPV1-442B) (Item Number B01.012.002):

This examination was limited to 31.00% aggregate coverage of the required weld volume. The principal limitation for this weld is the location of the long seam weld between the two outlet nozzles. The close proximity of the long seam weld to the outlet nozzles physically limits examination of the weld in the area between the nozzles. The specific limitations are identified on drawing 1213926D.

Reactor Vessel Upper Shell Long Seam Weld at 300 Degree (1RPV1-442C) (Item Number B01.012.003):

This examination was limited to 90.00% aggregate coverage of the required weld volume. The principal limitation for this weld involves nozzle interference which divides the weld into two parts. The Reactor Vessel Inlet Nozzle configuration prevents examination of the weld at the intersection of the weld and nozzle. The specific limitations are identified on drawing 1213926D.

Reactor Vessel Lower Shell Long Seam Weld at 60 Degree(1RPV3-442A) (Item Number B01.012.007):

This examination was limited to 84.00% argregate coverage of the required weld volume. The principal limitation for this weld is the presence of the core guide lugs welded to the vessel ID above weld 10-442 on the lower shell section. These lugs are approximately 19 inches tall, 27 inches wide, extend 8 inches radially toward the center of the vessel, and are positioned every 60 Degrees around the vessel circumference. The dimensions include the attachment weld radius. Because of their location, the lugs interfered with the axial and circumferential weld scans.

Reactor Vessel Lower Shell Long Seam Weld at 180 Degree(1RPV3-442B) (Item Number B01.012.008):

This examination was limited to 84.00% aggregate coverage of the required weld volume. The principal limitation for this weld is the presence of the core guide lugs welded to the vessel ID above weld 10-442 on the lower shell section. These lugs are approximately 19 inches tall, 27 inches wide, and extend 8 inches radially toward the center of the vessel, and are positioned every 60 Degrees around the vessel circumference. The dimensions include the attachment weld radius, because of their location these lugs interfered with the axial and circumferential weld scans.

Reactor Vessel Lower Shell Long Seam Weld at 300 Degree(1RPV3-442C) (Item Number B01.012.009):

This examination was limited to 84.00% aggregate coverage of the required weld volume. The principal limitation for this weld is the presence of the core guide lugs welded to the vessel ID above weld 10-442 on the lower shell section. These lugs are approximately 19 inches tall, 27 inches wide, and extend 8 inches radially toward the center of the vessel, and are positioned every 60 Degrees around the vessel circumference. The dimensions include the attachment weld radius, because of their location these lugs interferred with the axial and circumferential weld scans.

Weld Examination Summary

The reactor vessel shell welds were examined from the vessel inside surface using automated ultrasonic examination equipment. The examinations were done with various contact head arrangements to optimize the maximum examination coverage. This allowed each transducer to scan as close as possible to any obstruction around the area examined. Although the coverage requirements of 10 CFR 50.55a(g)(6)(ii)(A) could not be met, the examinations were performed with modified equipment and tooling designed to obtain the maximum coverage possible.

As a result of inspections performed, the 100% requirement has been determined to create a hardship for McGuire Nuclear Station. The reactor vessel welds were examined to the maximum extent practical to the requirements of Section V, Article 4 of the 1980 Edition through the Winter 1980 Addenda of the ASME Boiler and Pressure Vessel Code and the additional requirements of Regulatory Guide 1.150. To meet the 10 CFR 50.55a(g)(6)(ii)(A)(2) examination coverage requirements, design modifications would be necessary to gain access to the welds in order to obtain complete coverage. The design modifications are impractical due to the vast scope of work that would be required. Imposition of this

requirement would cause a considerable burden on Duke Energy with no commensurate safety benefit realized.

V. Alternate Examinations:

In addition to the volumetric examination that has been performed on the McGuire reactor vessel, Duke Energy has performed a visual examination of the internals and the inside of the reactor vessel as required by ASME Section XI, Table IWB-2500-1. This visual examination did not identify any rejectable conditions per ASME Section XI acceptance standards.

The use of radiography as an alternate volumetric examination method is not feasible due to component thickness and restrictions from physical barriers which prohibit access for the placement of source, image quality indicators, film, etc. In addition, the background radiation levels would not allow for a radiographic examination that would render meaningful results.

Performing the ultrasonic examination from the outside of the reactor vessel is not a viable option. The design of McGuire's reactor building prohibits access for the equipment and personnel from outside the vessel.

Duke Energy Corporation will continue to perform ultrasonic examinations of all reactor vessel welds to the maximum extent practical in accordance with the requirements of ASME Section V, Article 4, 1989 Edition and Regulatory Guide 1.150, Revision 1, Appendix A. The application of Code Case N-460 will be utilized in all cases where less than 100% but greater than 90% weld coverage is obtained. In cases where weld coverage of 90% or less is obtained, a request for relief from ASME Section XI Code requirements will be submitted.

Duke Energy Corporation proposes as an alternative to the greater than 90% coverage requirement of 10 CFR 50.55a(g)(6)(ii)(A), that the examination coverage obtained on the welds listed in Attachment 1 be considered to provide an acceptable level of quality and safety.

No additional examinations will be required.

VI. Justification for Granting Alternative:

10 CFR 50.55a(A)(5) states that "Licensees that make a determination that they are unable to completely satisfy the requirements for the augmented reactor vessel shell weld examination specified in 50.55a(g)(6)(ii)(A) shall submit information to the Commission to support the determination and shall propose an alternative to the examination requirements that would provide an acceptable level of quality and safety. 10 CFR 50.55a(a)(3) states that alternatives to the requirements of paragraph (g) may be used when authorized by the Director of the Office of Nuclear Reactor Regulation. The proposed alternative(s) must demonstrate that an acceptable level of quality and safety, or compliance with the specified

requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.

Examination of 100% of reactor pressure vessel shell welds would result in hardship and unusual difficulty without compensatory increase in safety being realized. Examination of the accessible weld volume provides sufficient and reasonable assurance of vessel integrity. The reduction in the expected examination coverage will not endanger life or property or the common defense and security because the reactor coolant system is designed and constructed to have a low probability of gross rupture or significant leakage throughout its design life. Technical Specifications 3/4.4.6 for McGuire Nuclear Station places conservative limits on the amount of reactor coolant leakage allowed during system operation. Any weld failure would allow additional coolant to leak from the system. The reactor coolant system leakage detection system is in place to detect any variation in the system water inventory. The purpose of the containment building is to retain any such leakage within its boundaries. If leakage exceeds Technical Specification 3.4.6.2, procedures are in place to assure safe shutdown of the unit within specified time limits.

Due to the design of the McGuire reactor vessels and location of the physical obstructions, it is not feasible to obtain the examination coverage required by 10 CFR 50.55a(g)(6)(ii)(A)(2) without placing undue hardship on Duke Energy. Based on the portions of the required volumetric and visual examination that have been completed, any existing pattern of degradation would have been detected.

Duke Energy Company will continue to ultrasonically examine the reactor vessel B1.10 category welds to the extent practical within the limits of original design and construction. This will provide reasonable assurance of weld/component integrity.

Pursuant to 10 CFR 50.55a(g)(6)(i), granting this alternative for the reactor vessel B1.10 category welds will provide reasonable assurance of weld/component integrity, "is authorized by law, and will not endanger life or property or the common defense and security and is other wise in the public interest giving due consideration to the burden upon the licensee that could result if the requirement were imposed on the facility."

VII. Implementation Schedule:

The reactor vessel welds listed above will be examined during the third period of the second interval. The examinations are currently scheduled during EOC 15.

Attachments:

- 1. Listing of information for welds with limited UT examination coverage
- 2. Detailed drawings of affected welds including calculation methods

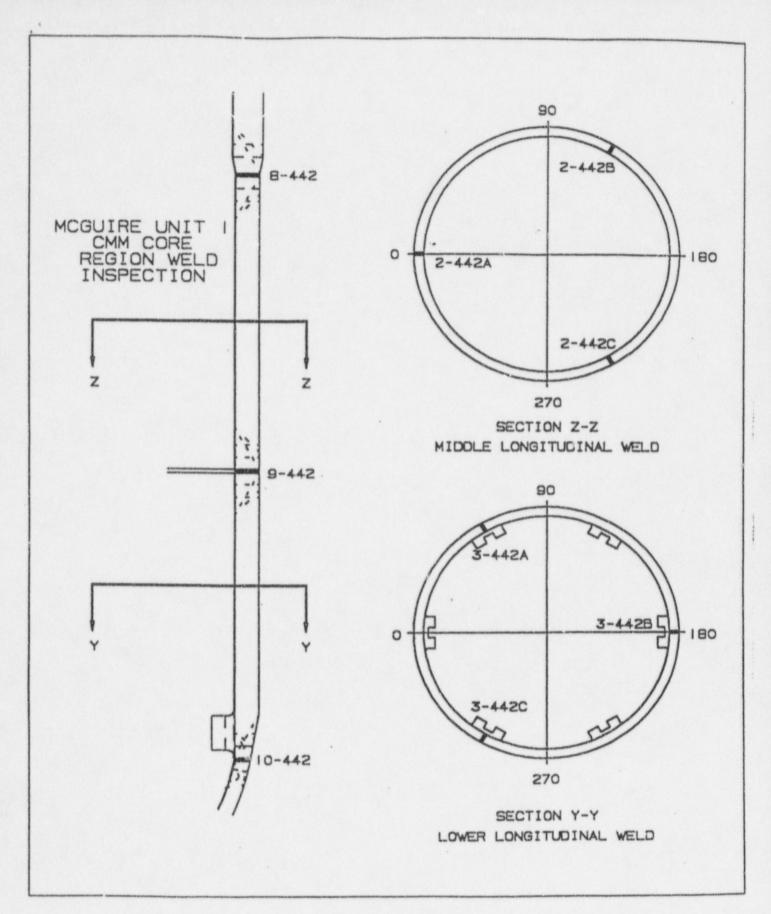
Reviewed By: Sarlwar Date: 7/9/97

Reviewed By: Sarlwar Date: 7/9/97

ASME Class 1 NDE Inservice Inspection Request For Alternative Serial No. 97-002 For McGuire Unit 1 Based On ASME XI - 1980 Code Through Winter 1980 Addenda

Attachment 1 Page 1 of 1

Item No.	Item No. Exam Category/ System Or Fu Figure No. Component		Function	Area To Be Examined	Reason For Request	Weld ID No.	Licensee Proposed Alternate Examination
B01.011.003	B-A IWB-2500-1	Reactor Vessel	Houses the fuel assemblies, control rods, and vessel internals, also directs the flow of reactor coolant	Lower Shell to Bottom Head Weld	Limited scan due to six core guide lug obstructions Actual coverage obtained = 59.00%	1RPV10-442	None
B01.012.001	B-A IWB-2500-2	Reactor Vessel	Houses the fuel assemblies, control rods, and vessel internals, also directs the flow of reactor coolant	Upper Shell Long Seam Weld at 60 Deg.	Limited scan due to nozzle geometric configuration. Actual coverage obtained = 90.00%	1RPV1-442A	None
B01.012.002	B-A IWB-2500-2	Reactor Vessel	Houses the fuel assemblies, control rods, and vessel internals, also directs the flow of reactor coolant	Upper Shell Long Seam Weld at 180 Deg.	Limited scan due to nozzle geometric configuration. Actual coverage obtained = 31.00%	1RPV1-442B	None
B01.012.003	B-A IWB-2500-2	Reactor Vessel	Houses the fuel assemblies, control rods, and vessel internals, also directs the flow of reactor coolant	Upper Shell Long Seam Weld at 300 Deg.	Limited scan due to nozzle geometric configuration. Actual coverage obtained = 90.09%	1RPV1-442C	None
B01.012.007	B-A IWB-2500-2	Reactor Vessel	Houses the fuel assemblies, control rods, and vessel internals, also directs the flow of reactor coolant	Lower Shell Long Seam Weld at 60 Deg.	Limited scan due to core guide lug obstruction Actual coverage obtained = 84.00%	1RPV3-442A	None
B01.012.008	B-A IWB-2500-2	Reactor Vessel	Houses the fuel assemblies, control rods, and vessel internals, also directs the flow of reactor coolant	Lower Shell Long Seam Weld at 180 Deg.	Limited scan due to core guide lug obstruction Actual coverage obtained = 84.00%	1RPV3-442B	None
B01.012.009	B-A IWB-2500-2	Reactor Vessel	Houses the fuel assemblies, control rods, and vessel internals, also directs the flow of reactor coolant	Lower Shell Long Seam Weld at 300 Deg.	Limited scan due to core guide lug obstruction Actual coverage obtained = 84.00%	1RPV3-442C	None



Vessel Weld Layout Figure 1

SER. NO. 97-002 ATTACHMENT 2 REACTOR VESSEL OUTLINE PAGE 1 OF 28

DUKE POWER COMPANY MC GUIRE UNIT 1 REACTOR VESSEL EXAMINATION

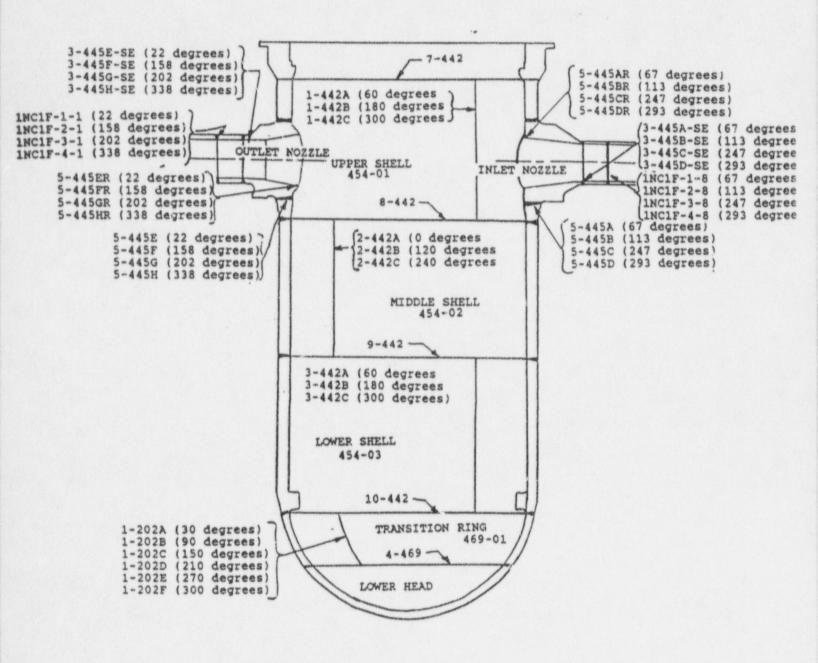
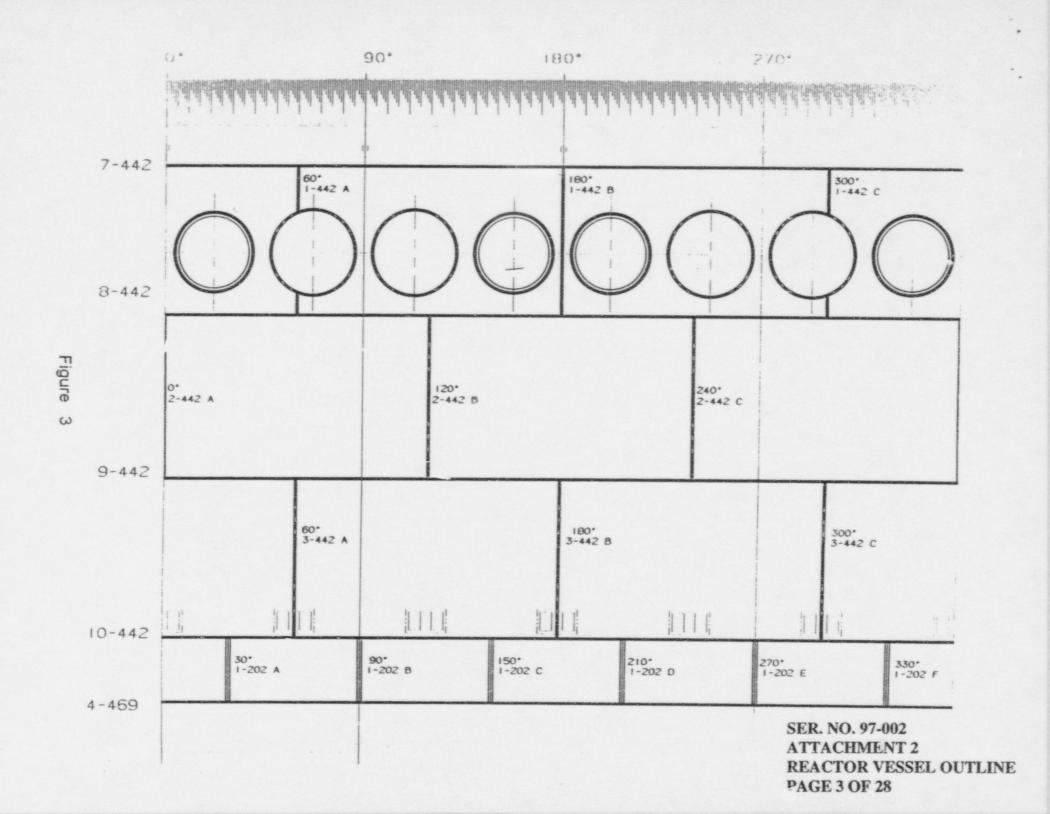


Figure 2



EXAMINATION COVERAGE FOR WELD:

LWR SHELL TO LWR HEAD CIRC WELD

AGGREGATE COVERAGE OBTAINED:

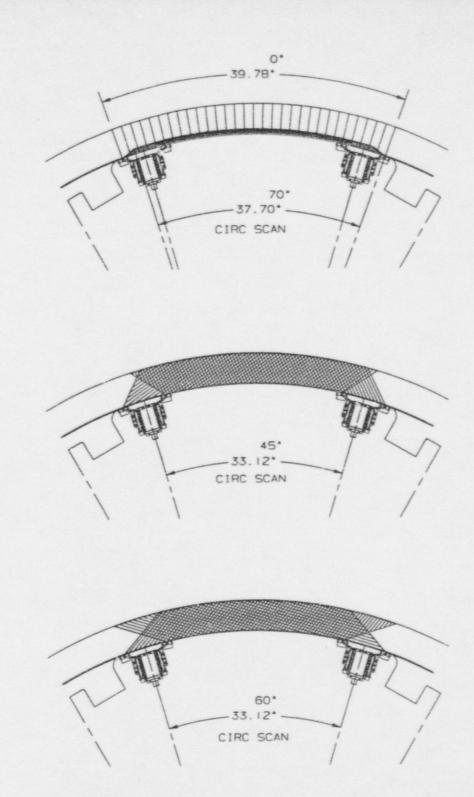
59%

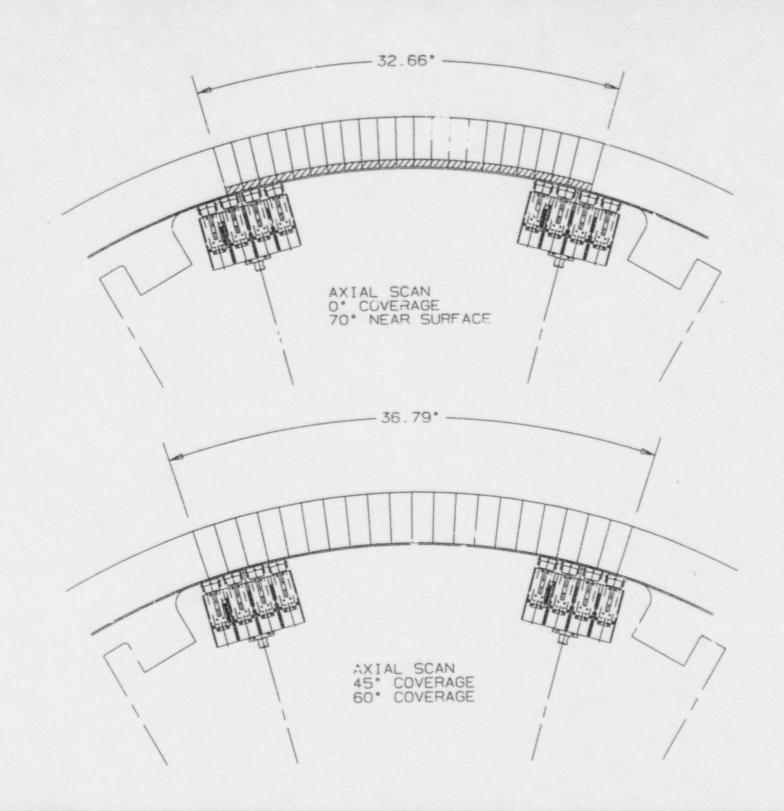
10-442

SCAN PLAN DRAWING: 1213929D-02
Zone Coverage Obtained

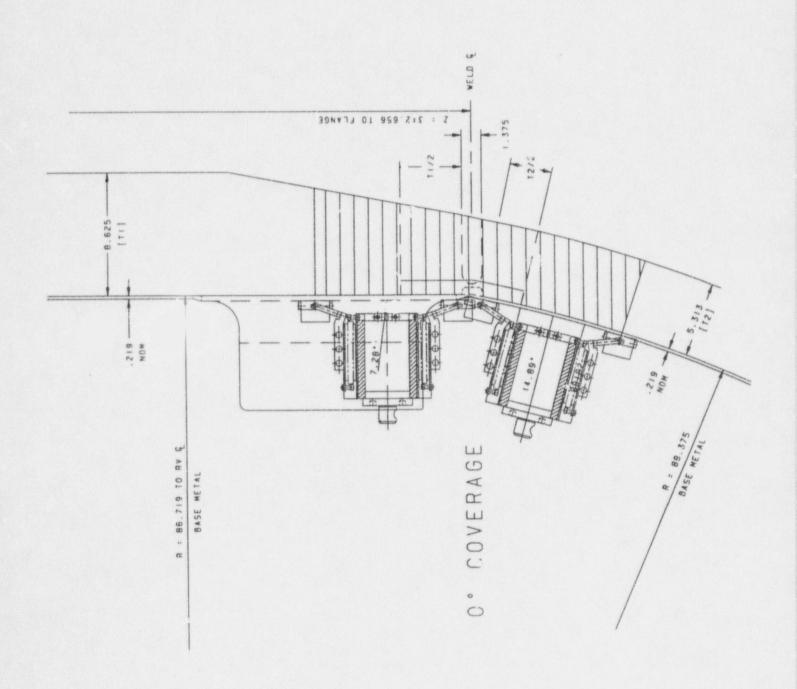
Zone Coverage Obtained									
Weld: 63% Adjacent Base					59%		Near (ID) Su	urface:	60%
Examination Volume Datinition									
-	A M		d Length:	544.872		V-1 C	-laulatian		
Mold	Area Meas		en in		Weld	Volume Ca	4091.989 c	u in-	
The state of the s					ace Metal	23908.98 c	research contract		
Near Surfa		8.58			Near Surfa		4675.002 c		
Weld	Americanicalis	MANAGES AND STREET	v∉Examin	ation Cove	rage, Calcu	lations	HIPADIAN)	CHANGE WINDS	SHAME CANAL
aveig	Exam.		Area	Length	Volume	\'olume			
	Angle	Beam			Examined	A 198 A	Percent		
Entry #	(deg.)	Direction	(sq. in.)	(in.)		(cu. in.)			
1	0	n/a	7.5	361.3	2713.0	2713.0	100%		
2	0	n/a	0.0	183.6	0.0	1379.0	0%		
3	45	1	7.5	334.1			100%		
4	45	2	7.5	334.1	2509.1	2509.1	100%		
5	45	1	0.0	210.8	0.0	1583.0	0%		
6	45	2	0.0	210.8	0.0	1583.0	0%		
7	45	3 4	7.5	300.3		2258.8	100%		
8	45	3	7.5	300.8	2258.8	2258.8	0%		
9	45 45	4	0.0	244.1 244.1	0.0	1833.2 1833.2	0%		
11	60	1	7.5			2509.1	100%		
12	60	2	7.5		2509.1	2509.1	100%		
13	60	1	7.5			1583.1	100%		
14	60		0.0			1583.1	0%		
15	60	2	7.5			2258.8	100%		
16	60	4	7.5			2258.8	100%		
17	60	3	0.0		0.0	1833.2	0%		
18	60	4	0.0		0.0	1833.2	0%		
Adjacent	Base Meta	ıl .		Totals:	23367.6	36828.4	63%	-	
rajacent	Exam.	"	Area	Length	Volume	Volume			
	Angle	Beam			Examined		Percent		
Entry #	(deg.)	Direction	(sq. in.)		(cu. in.)		Examined		
1	0	n/a	43.9				100%		
2	0	n/a	0.0	183.5			0%		
3	45	18.2	43.9						
4	45	18.2	0.0						
5	45	3	43.9						
6	45	4	43.9			13197.8	100%		
7	45	3	0.0		0.0	10711.1	0%		
8 9	45	4	0.0		0.0	10711.1	0% 100%		
10	60	182	43.9						
11	60 60	18.2	43.9						
12	60	4	43.9				100%		
13	60	3	0.0				0%		
14	60	4	0.0				0%		
	,			Totals	97963.4	167364.0	59%		
Near Sur			A	Longth	Values	Volume			
1	Exam.	Been	Area	Length	Volume	Volume	Percent		
Entore	Angle (dea)	Beam			Examined		Examined		
Entry #	(deg.) 70	Direction	(sq. in.)	(in.) 3 296.6	(cu. in.) 2544.8				
2	70	axial	0.0						
3	70	circ	8.6						
4	70	circ.	0.0						
				Totals					

SER. NO. 97-002 ATTACHMENT 2 B01.011.003 1RPV10-442 PAGE 4 OF 28

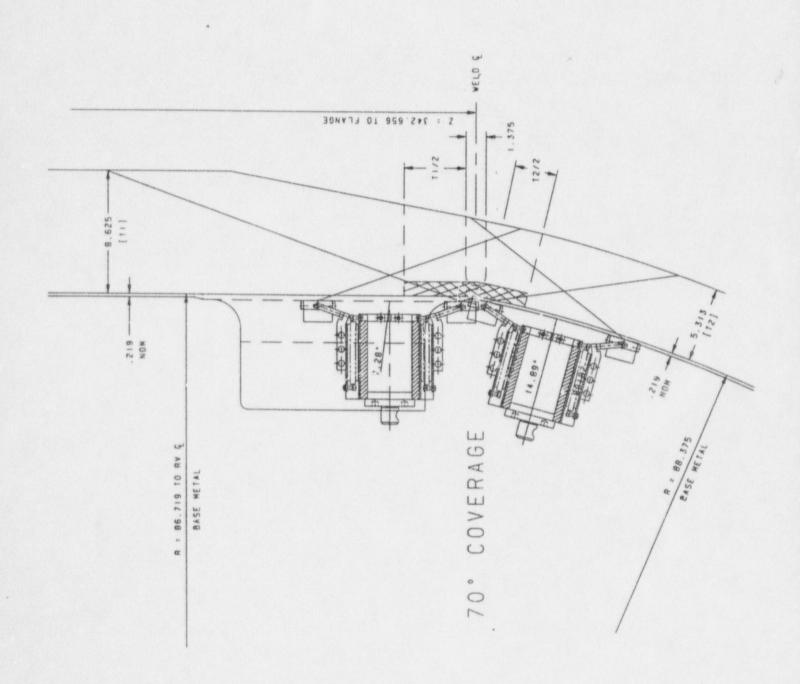




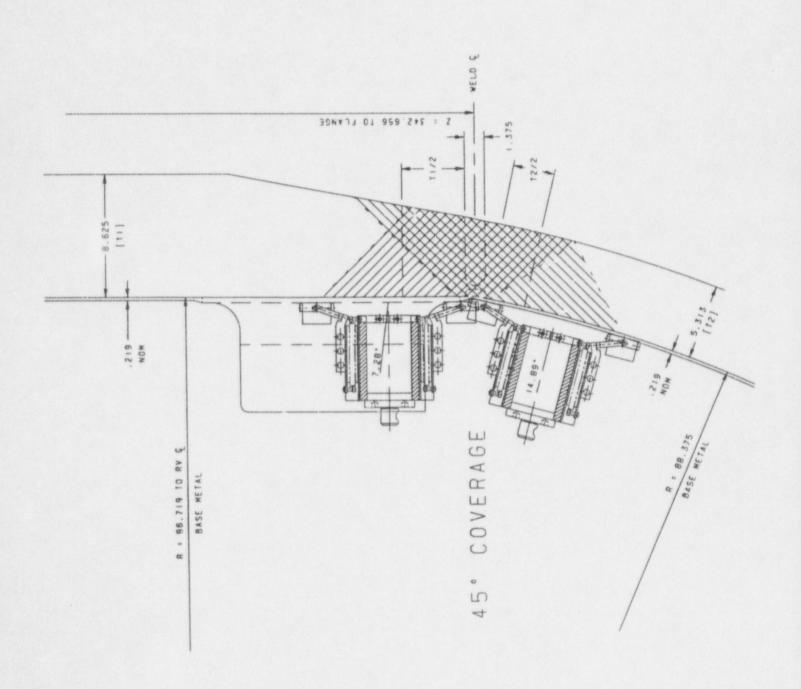
SER. NO. 97-002 ATTACHMENT 2 B01.011.003 1RPV10-442 PAGE 6 OF 28



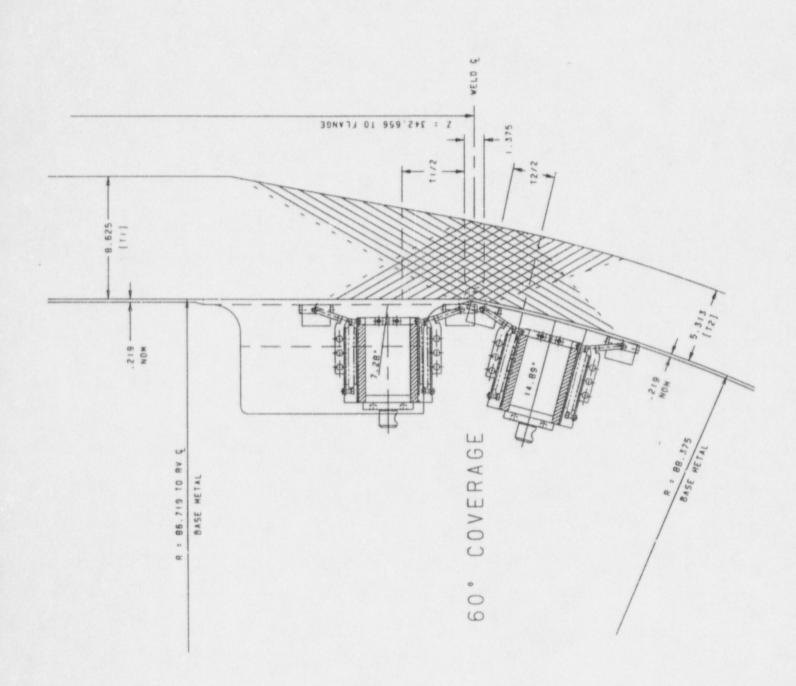
SEF. NO. 97-002 AT CHMENT 2 B01.541.003 1RPV10-442 PAGE 7 OF 28



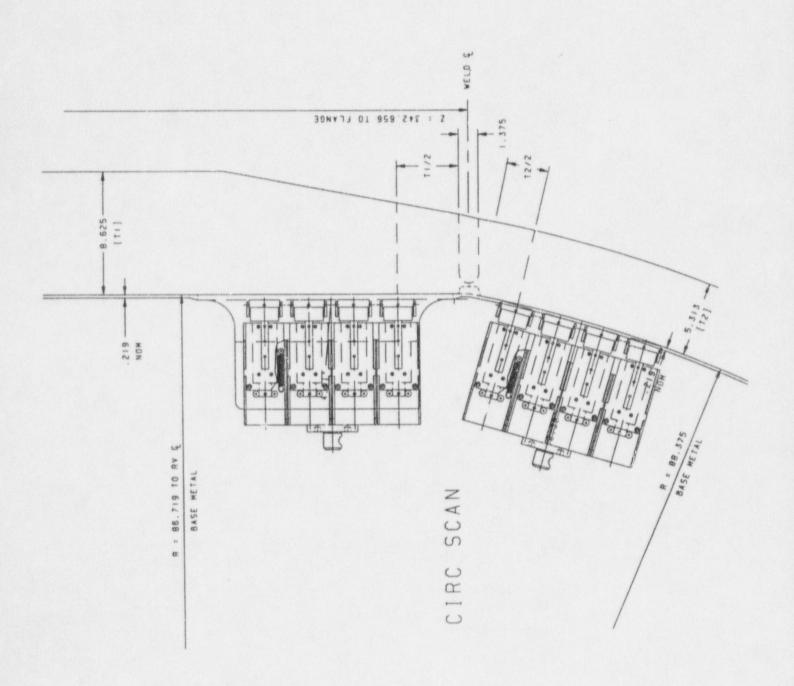
SER. NO. 97-002 ATTACHMENT 2 B01.011.003 1RPV10-442 PAGE 8 OF 28



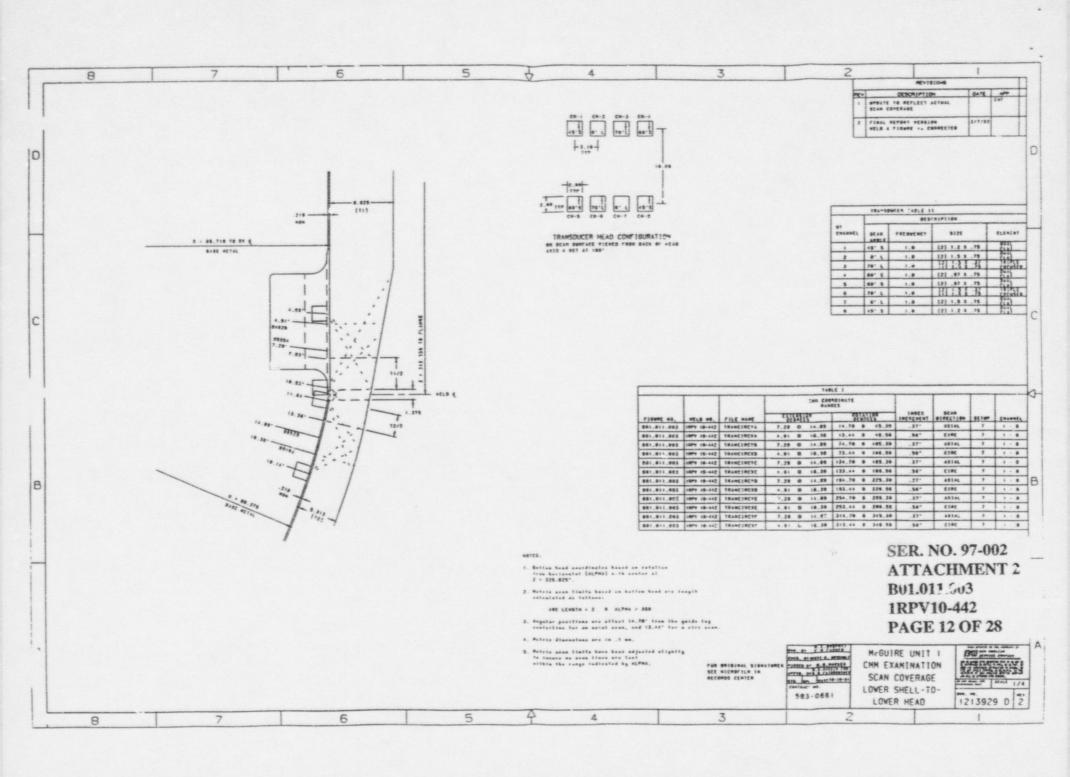
SER. NO. 97-002 ATTACHMENT 2 B01.011.003 1RPV10-442 PAGE 9 OF 28



SER. NO. 97-002 ATTACHMENT 2 B01.011.003 1RPV10-442 PAGE 10 OF 28



SER. NO. 97-002 ATTACHMENT 2 B01.011.003 1RPV10-442 PAGE 11 OF 28



EXAMINATION COVERAGE FOR WELD:

UPPER LONGITUDINAL WELD

AGGREGATE COVERAGE OBTAINED:

90%

1-442 A,C

SCAN PLAN DRAWING: 1213936D-02

	British Book Street Control of Street	OUNITE	PARTY NAMED AND POST OFFICE ADDRESS OF THE	e Covera	SHOW AND A PARTY OF THE PARTY NAMED IN THE PARTY NA	lned.			The second second second second
Weld:			Near (ID						
PENSIFE.	THE PROPERTY OF THE PARTY OF TH	A	Exam	ination	lume Defi	nltlon	HOO WHOM	Brichte.	E WARRANGE
	Area Ster	surement	eld Length:	24.64	in.		-		
Weld	Area mea		on in		14/-14	Volume (Calculatio		
	Base Met	124.70	sq. in.		Weld	D M-1-	364.179		
Near Su	face Met		sq. in.			Base Meta			
rear Sur	Tave	12.23	sq. in.		Near Surf	ace	301.3472	2 cu. in	
₩275€ Weld	atribació de	DAME OF THE	naExamin	ation Gove	rage Calc	ulations	the same of	AR ATTAIN :	har elected a
AAGIO	Exam.		A	Lanath	Maluma	\/-b			
	Angle	Room	Area	Length	Volume	Volume			
Entry #	-	Beam		Examined					
Entry #	(deg.)	Direction	(sq. in.)			(cu. in.)		d	
2	0	n/a	14.8						
3	45	1	14.8						
4	45	2	14.8				100%		
-	45	1	0.0						
5	45	2	0.0						
6	45	3	14.8		364.2				
	45	4	14.8						
8	60	1	14.8						
9	60	2	14.8						
10	60	1	0.0			87.6			
11	60	2	0.0			87.6			
12	60	3	14.8				100%		
13	60	4	14.8				100%		
A -41		-	WELL SHOULD AND AND ADDRESS OF THE PARTY OF	Totals:	2927.0	3277.6	89%		
Adjacent	Base Met	tal							
	Exam.	_	Area	Length	Volume	Volume			
	Angle	Beam		Examined					
Entry #	(deg.)	Direction	(sq. in.)		(cu. in.)			1	
1	0	n/a	124.1						
2	45	1&2	115.7						
3	45	182	0.0	5.9	0.0	735.8			
4	45	3	124.1	24.6	3057.3	3057.3	100%		
5	45	4	124.1	24.6	3057.3	3057.3	100%		
6	60	1&2	115.7	18.7	2164.0	2321.5	93%		
7	60	1&2	0.0	5.9	0.0	735.8	0%		
8	60	3	115.7	24.6	2849.9	3057.3	93%		
9	60	4	115.7	24.6	2849.9	3057.3	93%		
Near Sur	face			Totals:	19199.7	21401.3	90%		
	Exam.		Area	Length	Volume	Volume			
	Angle	Beam		Examined			Paraant		
Entry #	(deg.)	Direction	(sq. in.)	(in.)	(cu. in.)	The second secon	Percent		
1		axial	12.2	18.7	228.8		Examined	,	
2		axial	0.0	5.9	0.0	228.8	100%		
3		circ.	12.2			72.5	0%		
		Oli U.	12.2	24.6	301.3	301.3	100%	SER.	NO. 97-00
	NAME OF TAXABLE PARTY.			Totals:	530.2	602.7	88%	MAJERA J	

SER. NG. 97-002 ATTACHMENT 2 B01.012.001, B01.012.003 1RPV1-442A, 1RPV1-442(PAGE 13 OF 28

EXAMINATION COVERAGE FOR WELD: 1-442 B

UPPER LONGITUDINAL WELD

AGGREGATE COVERAGE OBTAINED:

31%

			ACRES CONTRACTOR AND ACCORDING TO						
Weld	: 28%	•		erage O lase Metal:			Near (ID) S	iurfece:	43%
medical engineers	2000 Ma	and the same of the	xaminatio	. Volume	Definition	Memory	or of the second	le article artis	5126,529
	rea Mea	Surement	d Length:	74.265		Volume C	alculation		
Veld	wea mea		sq. in.		Weld	TOTAL C	1097.637	cu. in.	
Adjacent Base Metal		124.08	45.77			ase Metal	9214.801		
Near Surface			sq. in.		Near Surfa		908.261		
2. 大型 大型 (1.5 · 1	CHI TONG BAR	soese Exa	mination	loverage (alculation	B TO TELEVISION	A CENSOR OF	N SHEET IN	Dr. Grain
Veld		Special Residence of the Section of	STREET, STREET		A THE ARMST THE STATE OF THE ST	CONTRACTOR STATEMENT OF THE STATEMENT OF	THE REAL PROPERTY OF THE PARTY OF THE PARTY OF	Dark States (Carlo Special States)	-
	Exam.		Area	Length	Volume	Volume			
	Angle	Beam			Examined				
Entry #	(deg.)	Direction	(sq. in.)	(in.)	(cu. in.)		Examined		
1	0 .	n/a	14.8	20.4	302.0	302.0	100%		
2 3	0	n/a	0.0	53.8	0.0	795.7			
4	45	1	14.8	20.4	302.0	302.0			
	45	1	0.0	53.8	0.0	795.7			
5 6	45	2	14.8	20.4	302.0	302.0			
7	45 45	2	0.0	53.8	0.0 302.0	795.7			
8	45	3	14.8	20.4 53.8	0.0	302.0 795.7			
9	45	4	14.8	20.4		302.0			
10	45	4	0.0	53.8	0.0	795.7			
11	60	1	14.8	20.4	302.0	302.0			
12	60	1	0.0	53.8	0.0	795.7			
13	60	2	14.8	20.4	302.0	302.0			
14	60	2 2	0.0	53.8	0.0	795.7			
15	60	3	14.8	20.4		302.0			
16	60	3	0.0	53.8	0.0	795.7			
17	60	4	14.8	20.4		302.0			
18	60	4	0.0		0.0	795.7			
				Totals:	2113.7	7683.5	28%		
Adjacent Base Meta									
	Exam.		Area	Length	Volume	Volume			
Cata: #	Angle	Beam			Examined				
Entry #	(deg.)	Direction	(sq. in.)	(in.)	(cu. in.)	(cu. in.)			
2	0	n/a	124.1						
2 3	0 45	n/a 18.2	0.0 124.1						
4	45	18.2	0.5	53.8					
	45	3	124.1	20.4	2535.0	2535.0			
5	45	3	0.0						
6 7	45	4	124.1			2535.0			
8	45	4	0.0						
9	60	182	124.1						
10	60	182	0.0						
11	60	3	124.1						
12	60	3	0.0						
13	60	4	124.1						
14	60	4	0.0	53.8	0.0	6679.8	0%		
Near Curtage				Totals	17744.7	57823.8	31%		*******
Near Surface	Even		A ====	Length	Volume	Volume			
	Exam.	Decree	Area		Examined		Percent		
Entry #	Angle	Beam			(cu. in.)	(cu. in.)			
Lildy #	(deg.) 70	Direction	(sq. in.)	(in.)					
2	70	axial	0.0						
2	70	circ	12.5						

249.9

0.0

499.7

20.4

53.8

Totals:

249.9

658.4

1158.1

100%

0%

43%

3

70

70

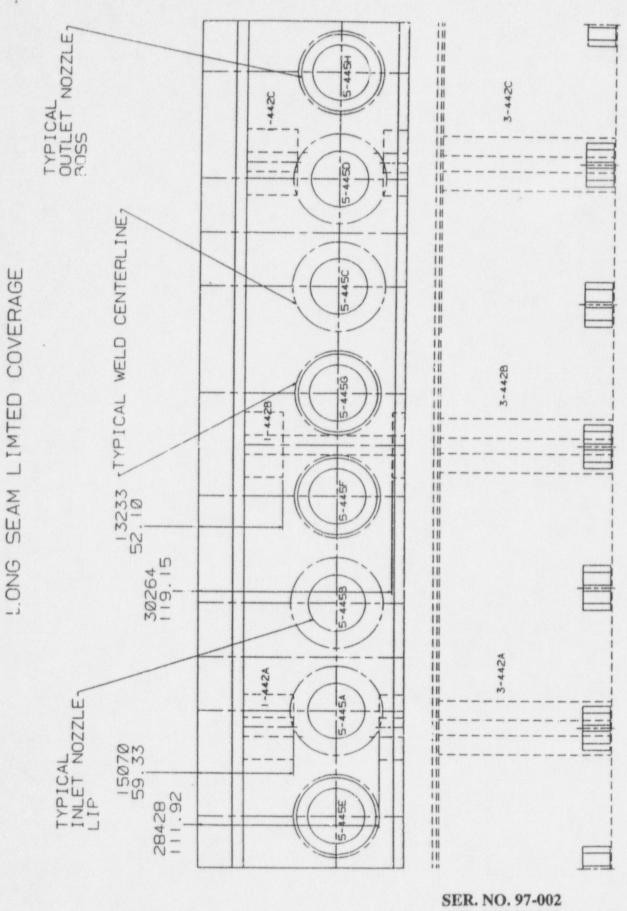
circ

circ

12.2

0.0

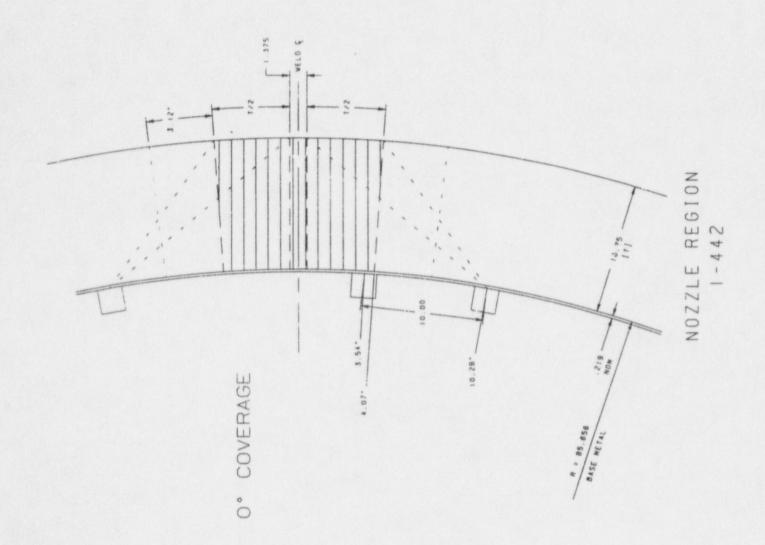
SER. NO. 97-002 ATTACHMENT 2 B01.012.002 1RPV1-442B PAGE 14 OF 28



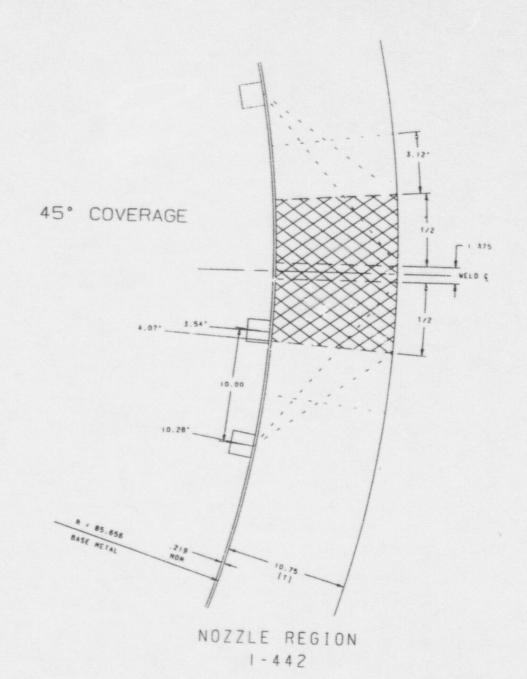
SER. NO. 97-002 ATTACHMENT 2 B01.012.001, B01.012.002, B01.012.003 1RPV1-442A, 1RPV1-442B, 1RPV1-442C PAGE 15 OF 28

				1 A	BLE I						
					ORDINATE NGES						
FIGURE NO.	FIGURE NO. WELD NO. FILE NAME			EXTENSION INCHES			TION	INCREMENT	SCAN	SETUP	CHANNE
801.012.005	IRPV 2-4428	MIDLONGIY	138.29	225.12	113.42	8	126.58	.37*	AXIAL	4	1 - 8
801.912.005	IRPV 2-4428	MIDLONGIX	136.87	226.48	114.70	8	125.30	.56*	CIRC	4	1 - 6
801.012.006	IRPV 2-442C	MIDLONG2Y	138.29	225.12	233.42	9	246.58	. 37*	AXIAL	4	1 8
801.012.006	IRPV 2-442C	MIDLONGZX	136.87	226.48	234.70	θ	245.30	.56*	CIRC	4	1 - 8
801.012.004	IRPV 2-442A	MIDLONG3Y	138.29	225.12	353.42	θ	366.58	.37*	AXIAL	4	1 - 8
801.012.004	IRPV 2-442A	MIDLONG3X	136.87	226.48	354.70	9	365.30	.56*	CIRC	4	1 - 8
801.012.001	IRPV 1-442A	UPPRLONGIYA	43.06	59.33	52.68	8	67.32	.37*	AX!AL	2	1 8
801.012.001	IRPV 1-442A	UPPRLONGIXA	38.90	59.33	53.09	θ	66.91	.56"	CIRC	2	1 - 8
801.012.001	IRPV 1-442A	UPPRLONGIYB	111.92	114.36	52.68	6	67.32	.37*	AXIAL	2	1 8
801.012.001	IRPV 1-442A	UPPRLONGIXB	111.92 2	116.13	53.09	θ	66.91	.56"	CIRC	2	1 - 8
801.012.002	IRPV 1-4428	UPPRLONGZYA	43.06 2	52.10	172.68	0	187.32	. 37*	AXIAL	2	1 - 8
301.012.002	IRPV 1-4428	UPPRLONG2XA	38.90 2	52.10	173.19	8	186.91	.56"	CIRC	2	1 8
801.012.003	IRPV 1-442C	UPPRLONG3YA	43.06 2	59.33	292.68	θ	307.32	.37*	AXIAL	2	1 - 8
801.012.003	IRPV 1-442C	UPPRLONG3XA	38.90 2	59.33	293.09	θ	306.91	.56"	CIRC	2	1 . 8
801.012.003	IRPV 1-442C	UPPRLONG3YB	111.92 2	114.36	292.68	0	307.32	. 37 *	AXIAL	2	1 - 8
801.012.003	1HPV 1-442C	UPPRLONG3XB	111.92 2	116.13	293.09	θ	306.91	.56*	CIRC	2	1 8
801.012.007	IRPV 3-442A	LOWRLONGIY	247.13 2	336.66	53.42	0	66.58	.37*	AXIAL	4	1 - 8
801.012.007	1RPV 3-442A	LOWRLONGIX	245.78 2	3.3.69	54.70	0	65.30	.56"	CIRC	4	1 - 8
301.012.008	IRPV 3-4428	LOWRLONGZY	247.13 2	6.66	173.42	θ	186.58	. 37*	AXIAL	4	1 - 8
301.012.008	IRPV 3-4428	LOWRLONGZX	245.78 2	333.69	174.70	θ	185.30	.56"	CIRC	4	1 - 8
301.012.009	IRPV 3-442C	LOSRLONG3Y	247.13 2	336.66	293.42	θ	306.58	. 37*	AXIAL	4	1 - 8
801.012.009	18PV 3-442C	LOWRLONG3X	245.78 2	333.69	294.70	8	305.30	.56"	CIRC	4	1 - 8

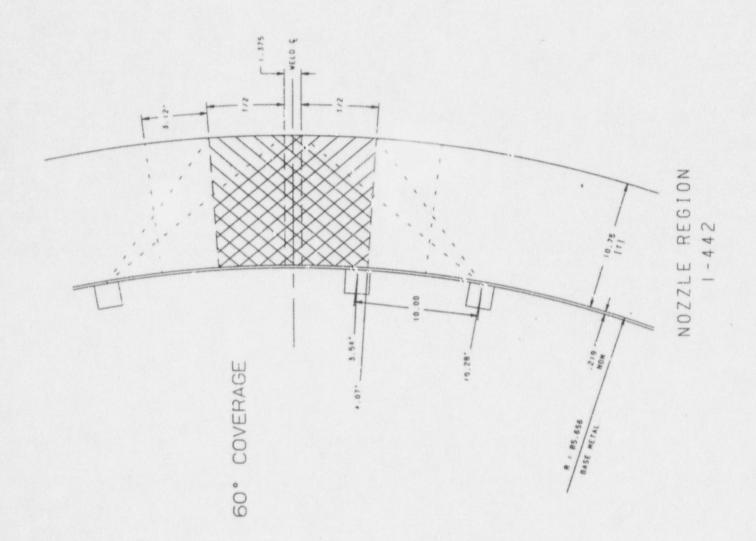
SER. NO. 97-002 ATTACHMENT 2 B01.012.001, B01.012.002, B01.012.003 1RPV1-442A, 1RPV1-442B, 1RPV1-442C PAGE 16 OF 28



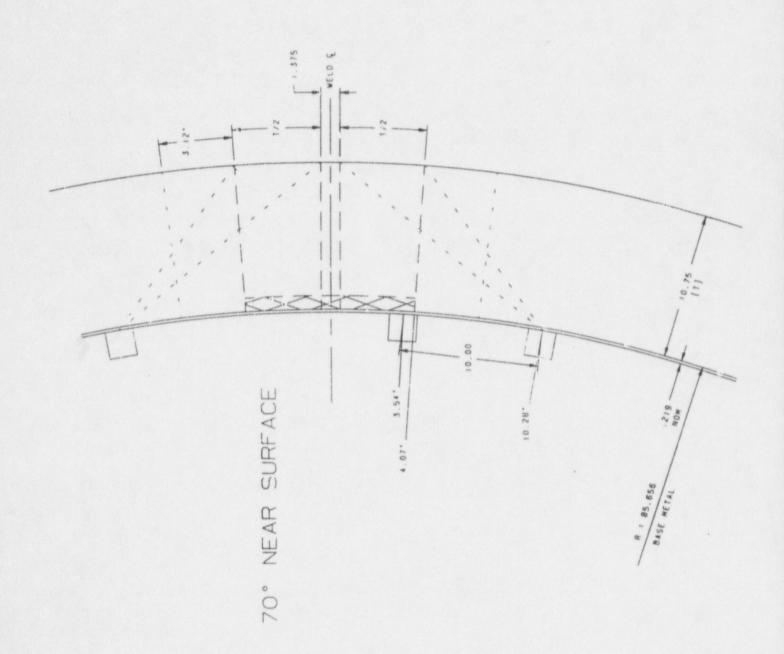
SER. NO. 97-002 ATTACHMENT 2 B01.012 001, B01.012.002, B01.012.003 1RPV1-442A, 1RPV1-442B, 1RPV1-442C



SER. NO. 97-002 ATTACHMENT 2 B01.012.001, B01.012.002, B01.012.003 1RPV1-442A, 1RPV1-442B, 1RPV1-442C PAGE 18 OF 28



SER. NO. 97-002 ATTACHMENT 2 B01.012.001, B01.012.002, B01.012.003 1RPV1-442A, 1RPV1-442B, 1RPV1-442C PAGE 19 OF 28



SER. NO. 97-002 ATTACHMENT 2 B01.012.001, B01.012.002, B01.012.003 1RPV1-442A, 1RPV1-442B, 1RPV1-442C PAGE 20 OF 28

EXAMINATION COVERAGE FOR WELD:

LOWER LONGITUDINAL WELD

3-442 A.B.C

AGGREGATE COVERAGE OBTAINED: 84% SCAN PLAN DRAWING: 1213926D-02 Zone Coverage Obtained Weld: 84% Adjacent Base Metal: 84% Near (ID) Surface: 84% A STATE OF THE STA Weld Length: 96.53 in. Area Measurement Volume Calculation 11.86 sq. in. Weld 1144.846 cu. in. Adjacent Base Metal 78.69 sq. in. Adjacent Base Metal 7595,946 cu. in. Near Surface 10.06 sq. in. Near Surface 971.0918 cu. in. A Section Coverage Calculations Coverage Calculations Weld Exam Area Length Volume Volume Angle Beam Examined Examined Required Percent Entry # (deg.) Direction (sq. in.) (in.) (cu. in.) (cu. in.) Examined 0 n/a 11.9 80.8 957.7 957.7 100% 2 0 n/a 0.0 15.8 0.0 187.2 0% 3 45 1 11.9 80.8 957.7 957.7 100% 4 45 2 11.9 80.8 957.7 957.7 100% 5 45 1 0.0 15.8 0.0 187.2 0% 6 45 2 0.0 15.8 0.0 187.2 0% 7 45 3 11.9 80.8 957.7 957.7 100% 8 45 4 11.9 80.8 957.7 957.7 100% 9 45 3 0.0 15.8 0.0 187.2 0% 10 45 4 0.0 15.8 0.0 187.2 0% 11 60 1 11.9 80.8 957.7 957.7 100% 12 60 2 11.9 80.8 957.7 957.7 100% 13 60 1 0.0 15.8 0.0 187.2 0% 2 14 60 0.0 15.8 0.0 187.2 0% 3 15 60 11.9 80.8 957.7 957.7 100% 16 60 4 11.9 957.7 80.8 957.7 100% 17 3 60 0.0 15.8 0.0 187.2 0% 18 60 4 0.0 15.8 0.0 187.2 0% Totals: 8619.3 10303.6 Adjacent Base Metal Exam. Area Length Volume Volume Angle Beam Examined Examined Required Percent Entry # (deg.) Direction (sq. in.) (in.) (cu. in.) (ou. in.) Examined 0 n/a 78.7 8.08 6354.2 6354.2 100% 2 0 0.0 1/8 15.8 0.0 1241.7 0% 3 45 182 78.7 80.8 6354.2 6354.2 100% 4 45 182 0.0 15.8 0.0 1241.7 0% 5 45 3 78.7 80.8 6354.2 6354.2 100% 6 45 4 78.7 80.8 6354.2 6354.2 100% 7 45 3 0.0 0.0 15.8 1241.7 0% 8 45 4 0.0 15.8 0.0 1241.7 0% 9 60 182 78.7 80.8 6354.2 6354.2 100% 10 60 182 0.0 15.8 0.0 1241.7 0% 11 78.7 60 3 80.8 6354.2 6354.2 100% 12 60 4 78.7 80.8 6354.2 6354.2 100% 13 60 3 0.0 15.8 0.0 1241.7 0% 14 60 4 0.0 15.8 0.0 1241.7 0% Totals: 44479.5 53171.6 Near Surface Area Length Volume Volume Angle Beam Examined Examined Required Percent Entry # (deg.) Direction (sq. in.) (cu. in.) (cu. in.) Examined (in.) 70 axial 10.1 80.8 812.3 812.3 100% 2 70 axial 0.0 15.8 0.0 0%

3

4

70

70

circ

circ.

10.1

0.0

80.8

15.8

Totals:

812.3

1624.7

0.0

158.7

812.3

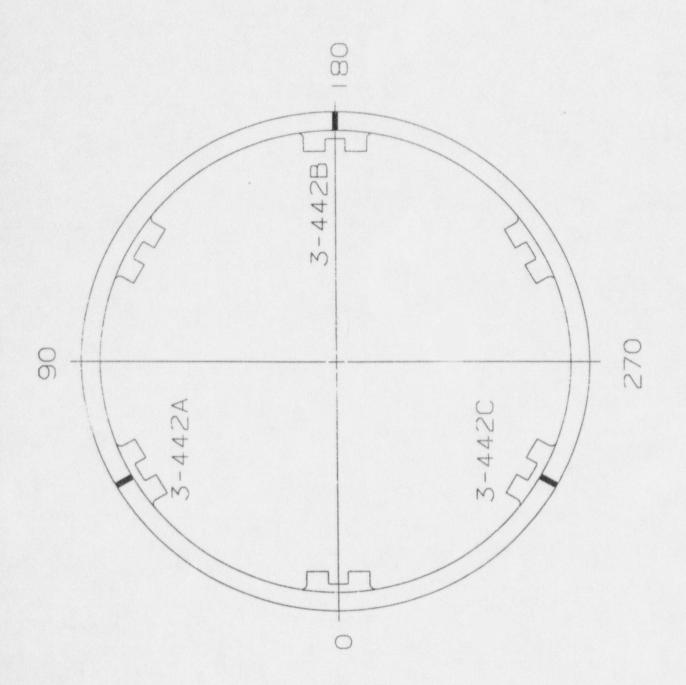
158.7

1942.2

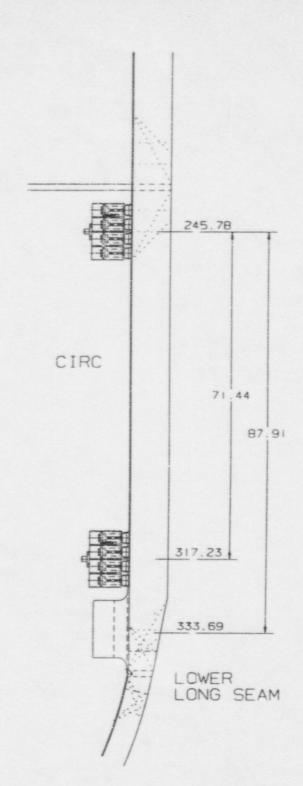
100%

0% R4%

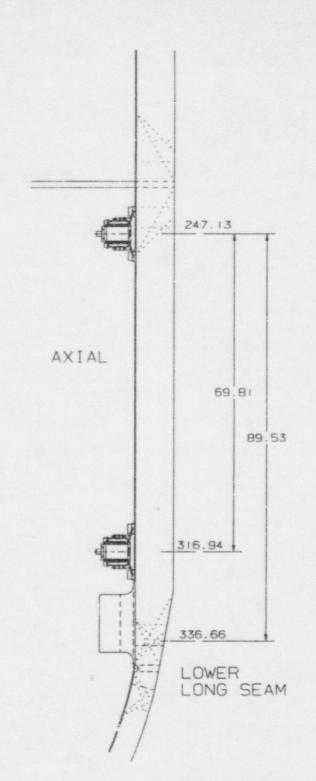
RPV3-442A, 1RPV3-442B, 1RPV3-442C B01.012.007, B01.012.008, B01.012.009 ATTACHMENT SER. NO. 97-002



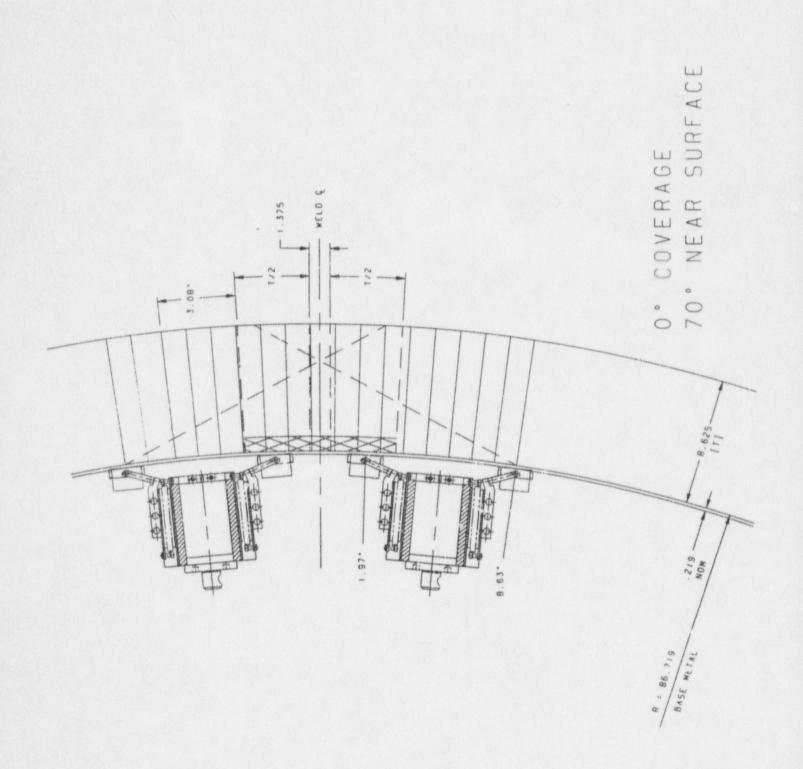
SER. NO. 97-002 ATTACHMENT 2 B01.012.007, B01.012.008, B01.012.009 1RPV3-442A, 1RPV3-442B, 1RPV3-442C PAGE 22 OF 28



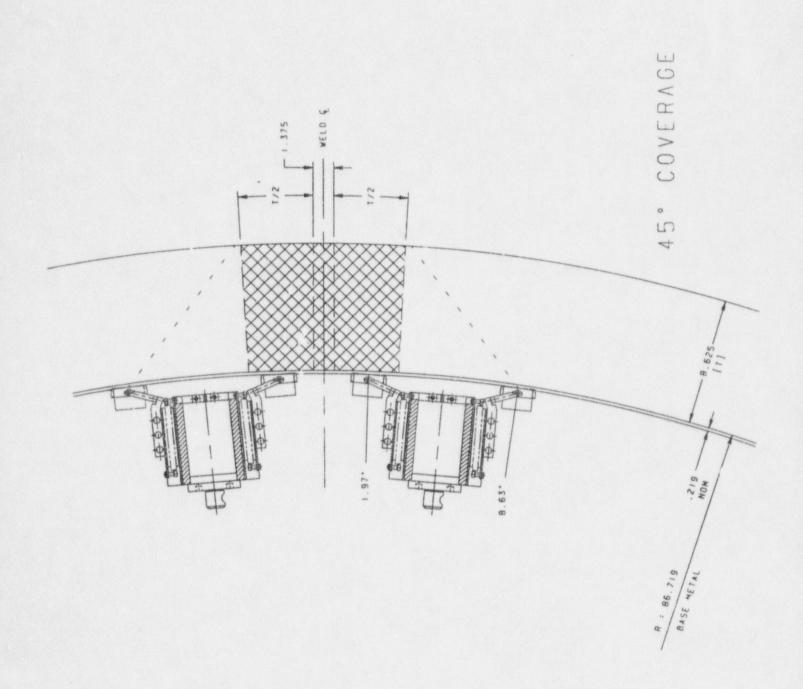
SER. NO. 97-002 ATTACHMENT 2 B01.012.007, B01.012.008, B01.012.009 IRPV3-442A, IRPV3-442B, IRPV3-442C PAGE 23 OF 28



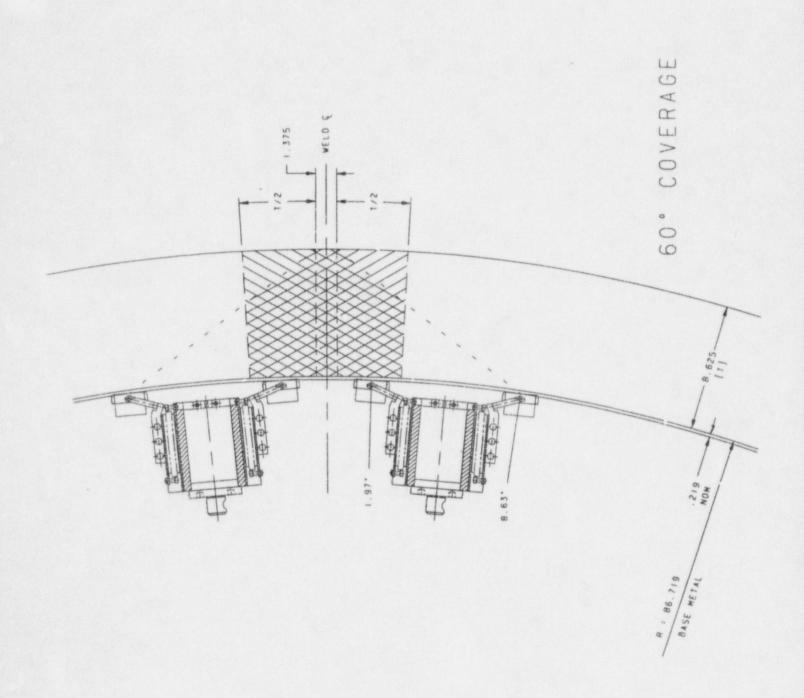
SER. NO. 97-002 ATTACHMENT 2 B01.012.007, B01.012.008, B01.012.009 1RPV3-442A, 1RPV3-442B, 1RPV3-442C PAGE 24 OF 28



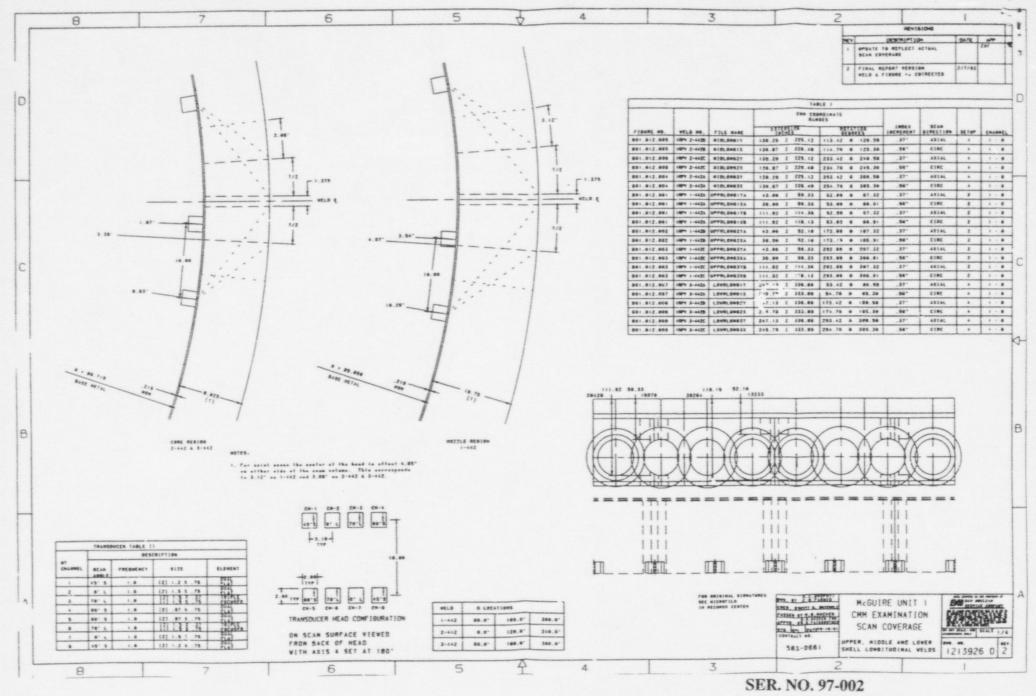
SER. NO. 97-002 ATTACHMENT 2 B01.012.007, B01.012.008, B01.012.009 1RPV3-442A, 1RPV3-442B, 1RPV3-442C PAGE 25 OF 28



SER. NO. 97-002 ATTACHMENT 2 B01.012.007, B01.012.008, B01.012.009 1RPV3-442A, 1RPV3-442B, 1RPV3-442C PAGE 26 OF 28



SER. NO. 97-002 ATTACHMENT 2 B01.012.007, B01.012.008, B01.012.009 1RPV3-442A, 1RPV3-442B, 1RPV3-442C PAGE 27 OF 28



SER. NO. 97-002 ATTACHMENT 2 B01.012.007, B01.012.008, B01.012.009 1RPV3-442A, 1RPV3-442B, 1RPV3-442C PAGE 28 OF 28