November 25, 1998

Carolina Power and Light Company ATTN: Mr. J. S. Keenan Vice President Brunswick Steam Electric Plant P. O. Box 10429 Southport, NC 28461

SUBJECT: INSPECTION PLAN - BRUNSWICK NUCLEAR PLANT

Dear Mr. Keenan:

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PDR

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On November 2, 1998, the NRC staff completed an inspection resource planning meeting. The staff conducted this review for all operating nuclear power plants in Region II to develop an integrated inspection plan. We conducted this meeting in lieu of the Semi-annual Plant Performance Review, which the staff has moved to February 1999, because of the agency's shift to an annual Senior Management Meeting cycle.

This letter advises you of our planned inspection effort resulting from the inspection planning meeting. We have provided it to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved before the inspector's arrival onsite. Enclosure 1 details our inspection plan for the next 4 months. We have provided the rationale or basis for each inspection outside the core inspection program so that you are aware of the reason for emphasis in these program areas. Resident inspections are not listed due to their ongoing and continuous nature.

During this scheduling cycle, we will continue to focus some of our discretionary inspection effort on the resolution of open inspection items. Therefore, we may conduct additional inspections, which are not listed on Enclosure 1, to close open inspection items that are ready to be resolved. We will notify you at least 3 weeks before the start of these inspections.

The NRC's general policy for reactor inspections is that we will announce each inspection, unless announcing the inspection could compromise the objectives of the inspectors. Therefore, we may not have included some specific inspections on Enclosure 1, such as in the security and radiological protection areas, and these inspections may not be announced.

Enclosure 2 contains a historical listing of plant issues, called the Plant Issues Matrix (PIM). The PIM includes only items from inspection reports or other docketed correspondence between the NRC and Brunswick Nuclear Plant. This material will be placed in the Public Document Room.

We will inform you of any changes to the enclosed inspection plan. If you have any questions, please contact me at (404) 562-4560.

Sincerely,

(Original signed by B. R. Bonser)

Brian R. Bonser, Chief Reactor Projects Branch 4 Division of Reactor Projects

Docket Nos. 50-325, 50-324 License Nos. DPR-71, DPR-62

Enclosures: 1. Inspection Plan 2. Plant Issues Matrix

cc w/encls: (See page 3)

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NRC Resident Inspector U. S. Nuclear Regulatory Commission 8470 River Road, SE Southport, NC 28461

	*FOR	PREVIOUS	CONCURRENCE -	SEE ATTACHED COPY
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### BRUNSWICK INSPECTION PLAN

INSPECTION PROCEDURE	TITLE/PROGRAM AREA	NO. OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
IP 84750/ IP 86750	Radiation Protection	1	11/16/98-11/20/98	Core Inspection
IP 82701	Emergency Preparedness	1	12/7/98-12/11/98	Core Inspection
IP 84750/ IP 86750	Radiation Protection	1	12/7/98-12/11/98	Core Inspection
IP 83750	Radiation Protection	1	1/4/99-1/8/99	Core Inspection
IP 40500	Effectiveness of Licensee Controls in Identifying, Resolving, and Preventing Problems	3	1/4/99-1/8/99	Core Inspection
IP 93809	SSEI - Safety System Engineering Inspection	5	1/11/99-1/15/99, 1/25/99-1/29/99	Core Inspection
IP 73753/ IP 92902	Maintenance	1	2/22/99-2/26/99	Regional Initiative - Open Item Followup and IGSCC/Internals Coating Inspection
IP 81700	Security	1	3/8/99-3/12/99	Core Inspection
IP 92904	Fire Protection	1	3/29/99-4/2/99	Regional Initiative - Open Item Followup and Fire Protection Improvement Program Review

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Region II BRUNSWICK

## United States Nuclear Regulatory Commission PLANT ISSUE MATRIX

Date: 11/25/1998 Time: 10:52:50

By Primary Functional Area

Date	Source	Functional Area	ID	Туре	Template Codes	Item Description
11/09/1998	1998009	Pri: ENG Sec:	NRC	POS	Pri: 58 Sec: Ter:	Improvements were noted in the evaluation of Inservice Testing Program non-conformances, identification of root and contributing causes, and implementation of corrective actions.
11/09/1998	1998009-04	Prí: ENG Sec:	NRC	VIOIV	Pri: 4B Sec: 2B Ter:	Review of a service water value test failure revealed an inadequate analysis justifying nuclear service water operability. The inspectors identified a violation for the licensee's failure to adequately identify potential program impacts resulting from this value test failure, as well as for the failure to obtain supervisory approval for the operability determination.
11/09/1998	1998009-01	Pri: ENG Sec: MAINT	NRC	VIOIV	Pri: 3A Sec: 3B Ter:	Three examples of failures to follow administrative procedures, and thus TS 5.4.1: 1) The inspectors identified a procedure violation during the performance of the Residual Heat Removal Remote Shutdown Panel System Flow Channel Calibration. Technicians did not properly flush the transmitter in accordance with the procedure. All other aspects of the evolution observed were performed satisfactorily. 2) The inspectors identified a violation for the licensee's failure to properly maintain or retain Inservice Testing Deviation Reports, which were a record of pump and valve testing performance problems and corrective actions. 3) The inspectors identified a procedural violation regarding the failure to perform a procedure revision prior to implementing a procedure acceptance criteria change for an Inservice Testing valve stroke time.
09/21/1998	1998009	Pri: ENG Sec:	NRC	NEG	Pri: 28 Sec: Ter:	Both channels of the radiation monitors for the steam jet air ejectors were declared inoperable as a result of conflicting terminology and data procluced by several different procedures. Resolution of the conflicting procedures, terminology, and data by the licensee continued to be poor.
08/29/1998	1998008	Pri: ENG Sec:	NRC	NEG	Pri: 4C Sec: Ter:	Long standing inspection findings identified between 1996 and 1997 involving potential discrepancies between plant as-found conditions and the plant licensing basis resulted in four violations being identified. However, for each of these violations, the NRC exercised discretion in accordance with the Enforcement Policy and refrained from issuing a citation.
08/29/1998	1998008-01	Pri: ENG Sec:	Licensee	NCV	Pri: 28 Sec: Ter:	The licensee identified a missed Technical Specifications surveillance associated with the Standby Gas Treatment System when they identified a deficiency in the aerosol mixing testing. This condition was treated as a Non-Cited Violation.
07/25/1998	1998008	Pri: ENG Sec:	NRC	POS	Pri: 5C Sec: Ter:	The licensee satisfactorily implemented the long-term solution for thermal hydraulic instabilities in response to NRC Generic Letter 94-02. Long-term Solutions and Upgrade of Interim Operating Recommendations for Thermal-Hydraulic Instabilities in Boiling Water Reactors.

Page: 2 of 4

Region II BRUNSWICK

# United States Nuclear Regulatory Commission PLANT ISSUE MATRIX

Date: 11/25/1998 Time: 10:52:50

By Primary Functional Area

Date	Source	Functional Area	ID	Туре	Template Codes	Item Description
09/30/1998	1998009	Pri: MAINT Sec:	NRC	POS	Pri: 3A Sec: Ter:	Corrective maintenance activities on the Service Water system were properly performed, adequately coordinated, and satisfactorily supervised.
08/19/1998	1998008	Pri: MAINT Sec:	NRC	NEG	Pri: 3A Sec: Ter:	Maintenance activities were conducted adequately with one exception inolving the failure to follow procedures after the identification of sluggish breaker handle response. This condition did not affect breaker operability and the licensee responded to the concern by initiating lessons learned training and appropriate corrective maintenance. This issue was identified as a minor violation.
08/19/1998	1998008	Pri: MAINT Sec:	NRC	NEG	Pri: 3A Sec: Ter:	Instrumentation and Control Technicians were generally knowledgeable and experienced in conducting electrical breaker preventive maintenance. Maintenance acitivities were conducted adequately with one exception involving the failure to follow procedures after the identification of sluggish breaker handle response. This condition did not affect breaker operability and the licensee responded to the concern by initiating lessons learned training and appropriate corrective maintenance. This issue was identified as a minor violation.
08/05/1998	1998008	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 5A Ter:	The Maintenance Rule expert panel conducted thorough reviews of system classification changes. The Plant Nuclear Safety Committee chairman provided a good safety focus and knowledge of industry maintenance rule experience.
11/09/1998	1998009	Pri: OPS Sec:	NRC	POS	Pri: 2A Sec: 3A Ter:	Increased licensee attention has resulted in a reduction in the number of "operator workarounds." However, opportunities to correct an outage-related workaround were not taken.
11/09/1998	1998009	Pri: OPS Sec:	NRC	POS	Pri: 3B Sec: 1C Ter:	The licensee's operator requalification program complied with the requirements and standards of plant procedures as well as the requirements of 10 CFR 55.59 for the areas inspected. The licensee developed and administered simulator examinations, in-plant Job Performance Measures (JPMs) and simulator JPMs that effectively identified areas in need of improvement.
11/09/1998	1998009	Pri: OPS Sec:	NRC	POS	Pri: 38 Sec: 1C Ter:	The feedback program and the remediai training programs were effective tools for improving the overall licensed operator requalification program.
07/30/1998	1998008	Pri: OPS Sec:	NRC	NEG	Pri: 38 Sec: Ter:	Due to the time constraints involved with the implementation of the Thermal Hydraulic Instability modification, the licensee did not meet their normal level of operator training expectations.

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Region II BRUNSWICK

## United States Nuclear Regulatory Commission PLANT ISSUE MATRIX

Date: 11/25/1998 Time: 10:52:50

By Primary Functional Area

Date	Source	Functional Area	ID	Туре	Template Codes	Item Description
07/30/1998	1998008	Pri: OPS Sec:	NRC	POS	Pri: 38 Sec: Ter:	Licensed simulator training was adequately performed. The operations crew responded satisfactorily to all the accident scenarios.
10/10/1998	1998009	Pri: PLTSUP Sec:	NRC	POS	Pri: 2A Sec: Ter:	Control of transient combustibles has improved. Two instances of improper storage of combustible materials were identified by the inspectors. Proper dispositioning of the items was satisfactorily accomplished.
08/29/1998	1998008	Pri: PLTSUP Sec:	NRC	NEG	Pri: 1C Sec: Ter:	A fire brigade drill program weakness was identified for not specifically demonstrating fire brigade response through drills in fire areas where deviations from NRC fire protection requirements had been approved.
08/29/1998	1998008	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	The implementation of procedural requirements for using and storing transient combustibles in safety-related areas was effective. Controls for combustible gas bulk storage and cutting and welding operations were being enforced.
08/29/1998	1998008	Pri: PLTSUP Sec:	NRC	POS	Pri: 2A Sec: Ter:	The observed level of plant housekeeping reflected good organization and cleanliness practices on the part of plant workers.
08/29/1998	1998008	Pri: PLTSUP Sec:	NRC	POS	Pri: 2A Sec: 2B Ter:	Corrective maintenance on degraded fire protection systems was accomplished in a timely manner. The maintenance and material condition of the fire protection equipment and features were satisfactory.
08/29/1998	1998008	Pri: PLTSUP Sec:	NRC	POS	Pri: 5A Sec: 5C Ter: 1C	The 1998 Nuclear Assessment Section assessment of the facility's fire protection program was comprehensive and was effective in identifying fire protection program programmatic deficiencies to management. The audit identified significant deficiencies in the past change management practices in the fire protection program. Planned corrective actions in response to the audit issues were substantial and included the initiation of a Fire Protection Program Upgrade Project.
08/29/1998	1998008	Pri: PLTSUP Sec: OPS	NRC	POS	Pri: 18 Sec: Ter:	The licensee's preparations for Hurricane Bonnie were timely and comprehensive. Corrective actions for plant restart were appropriate.

By Primary Functional Area

pe Codes:	Template Codes:	Functional Areas:
Drawn         DR         Construction         EV         Deviation         EV         Escalated Enforcement Item         Inspector follow-up item         ER         Licensee Event Report         C         Licensing Issue         IISC         Minor Violation         CV         NonCited Violation         EG	<ul> <li>Normal Operations</li> <li>1B Operations During Transients</li> <li>1C Programs and Processes</li> <li>2A Equipment Condition</li> <li>2B Programs and Processes</li> <li>3A Work Performance</li> <li>3B KSA</li> <li>3C Work Environment</li> <li>4A Design</li> <li>4B Engineering Support</li> <li>4C Programs and Processes</li> </ul>	OPS Operations MAINT Maintenance ENG Engineering PLTSUP Plant Support OTHER Other
OED Notice of Enforcement Discretion ON Notice of Non-Conformance 21 Part 21 OS Positive GI Safeguard Event Report R Strength	5A Identification 5B Analysis 5C Resolution ID Codes: NRC NRC Self Self-Revealed	

EEIs are apparent violations of NRC Requirements that are being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action" (Enforcement Policy), NUFEG-1600. However, the NRC has not reached its final enforcement decision on the issues identified by the EEIs and the PIM entiries may be modified when the final decisions are made.

URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. A URI may also be a potential violation that is not likely to be considered for escalated enforcement action. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.



# United States Nuclear Regulatory Commission PLANT ISSUES MATRIX

by SALP Functional Area

BRUNSWICK

25-Nov-98

DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)	SMM CODE
OPERATI	ONS					
7/2/98	Positive		IR 98-07 (O7.1)	NRC	The licensee conducted thorough reviews of Improved Technical Specifications implementation readiness during several Plant Nuclear Safety Committee meetings.	12345
6/29/98	URI		IR 98-07 (O2.1)	NRC	UFSAR discrepancies - Temps, in excess of UFSAR limits identified for the supply to RBCCW and the area around the Reactor Recirculation motors. The URI was issued to track the resolution of the elevated temperatures.	12345 A 00000
6/40/08	Ctronaile		10 00 07 (07 0)			B 00000
, ,	Strength		IN 98-07 (07.2)	NHC	The same level of control and risk assessment was applied to a short maintenance outage as a normal refueling outage.	12345 A 🖾 🗆 🗆 🗆 🗆 B 🗆 🗆 🗆 🗆
6/6/98	NCV		IR 98-06 (O8.1)	LICENSEE	A Non-Cited Violation was identfield for missed operator rounds.	C C C C C C C C C C C C C C C C C C C
5/21/98	Positive		IR 98-06 (07.1)	NRC	The restart affirmation conducted by the Plant Nuclear Safety Committee was a satisfactory evaluation of the overall site organization's readiness to restart Unit 1.	C D D D D D D D D D D D D D D D D D D D
5/20/98	Positive		IR 98-06 (O2.1)	NRC	Drywell housekeeping was effective in maintaining areas free of foreign material. Electrical and mechanical components were secured to support plant restart.	C U U U 1 2 3 4 5 A O O O O O B O O O O O C O O O O O

BRUNSWICK

25-Nov-98

DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SM	MC	ODES
5/9/98	Positive		IR 98-06 (O2.2)	NRC	The Torus was very clean, with only minor foreign material present; this material was removed during the licensee's closeout inspection. No deficiencies were identified during the forus sustain strainer modification installations.		1 2	3	4 5
					ouning the folds such of strainer mounication installations,	A			
						B			
5/7/98	Negative		IR 98-06 (O2.5)	NRC	The loss of the Supplemental Spent Fuel Cooling System on May 7, 1998, resulted	C	1 2	2	
					in a 9.5 degree Fahrenheit increase in fuel pool temperature. Eight to nine trips of the system occurred within a 3-week period.				
						A			
						B			
4/30/98	Positive		IR 98-06 (O1.2)	NRC	The core alterations conducted during the Unit 1 Refueling Outage were generally		1 2	3	4 5
					conducted according to procedure, had adequate communications, and were effectively controlied. However, the inspector noted that one fuel movement				
					occurred without the required communications: this deficiency was promptly corrected by a licensed operator in the Control Room	R			
						C			
4/26/98	/26/98 Negative		LER 98-02-00	LICENSEE	While in refueling mode, switch not locked in the Refuel position as required by TS		1 2	3 4	4 5
					with suppression poor inoperable.	۵			
						B			
						С			
4/25/98	Positive		IR 98-06 (O1.1)	NRC	Operator briefings, plant control, and response during the Unit 1 shutdown for the refueling outage were effective.		1 2	3 4	4 5
						A			
						в			
2/21/00	100					С			
3/31/98	VIO		IR 98-05 (01.1)	NRC	Scaffolding erected too close to safety equipment contrary to procedure requirements.		1 2	3 4	5
						A			
						в			
3/20/98	Positivo		ID 08 02 (04 1)	NDC		С			
0120100	1 031076		IN 90-02 (04.1)	NHG	Licensed operators had a good understanding of the Maintenance Rule, and understood their responsibilities for implementing the Maintenance Rule.		1 2	3 4	5
				, source and the maintenance main.	A				
						B			
						C			

	-	-		-	BRUNSWICK		25-Nov-98
DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
3/4/98	Negative		IR 98-03 (O2.3)	NRC	The diesel generator electrical lineups and plant drawings were consistent with the distribution panels. Four breakers were incorrectly labeled as compartment heaters instead of spares.	A B	
2/26/98	Negative		IR 98-03 (O1.2)	NRC	Current procedures do not control the use of the Emergency Rod in Notch Override switch. The licensee was responsive to this issue.	C A B	
2/25/98	Weakness		IR 98-03 (O1.1)	NRC	HPCI system was in standby and the material condition of the system was satisfactory. A weakness was identified in the licensee configuration control program for not identifying a valve installed on the HPCI turbine stop valve operating cylinder.	C A B	
2/12/98	Positive		IR 98-03 (O5.1)	NRC	An exercise scenario and a subsequent critique and training on the plant simulator which required an operations crew to demonstrate their ability to respond to plant transients and don self-contained breathing apparatus while the control room emergency ventilation system was undergoing modifications was performed in an effective and thorough manner.	C A B	
1/27/98	Positive		IR 97-15 (O1.1)	NRC	A Recirculation Pump Trip - occurred due to electrical fault on a transmission line. Operations response to the transient was good.	C A B	
12/27/97	Weakness		IR 97-13 (O8.4)	LICENSEE	DG Inoperable - Weakness - during bus maintenance tagout for DG did not provide clear instructions.	C A B	
12/27/97	VIO		IR 97-13 (07.2)	NRC	Retention of Required QA Records - Clearance sheets not maintained as QA records.	ABC	

-			-		BRUNSWICK		25-Nov-98
DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
12/27/97	Positive		IR 97-13 (O1.1)	NRC	Cold Weather Preparation - Program satisfactorily implemented. Procedures were adequate for contingency plans and operation checks for proper operation of the systems.	AB	
11/12/97	Positive		IR 97-13 (07.3)	NRC	Control of Mid-Cycle Outage - Excellent control - same process as regular refueling outage.	C	
11/12/97	Positive		IR 97-13 (07.1)	NRC	Safety Committee Review - Effective review of Unit 1 readiness for restart following mid-cycle outage.	BC	
11/6/97	MISC		IR 97-13 (O8.3)	LICENSEE	Recirculation Pump Runbacks - Cause determined to be improper functioning of level contacts in logic circuitry. Unit 2 had modification, Unit 1 next outage.	A B C	
10/29/97	Positive		IR 97-12 (O2.4)	NRC	Good Operation Response to a Transient - Reactor pump control linkage failed causing slow loss of pump without aiarm, operator caught quickly.	B C A	
10/24/97	NCV		IR 97-12 (O4.2)	LICENSEE	Control Rod Movement Error - Operator inserted rods further than allowed. Weakness in direction provided for reactivity manipulation control.	B C A	
10/12/97	Positive		IR 97-12 (O2.2)	NRC	Drywell Inspection - Drywell free of foreign material and ready for closeout.	B C A B	
						С	

-					BRUNSWICK	25-Nov-98
DATE	TYPE(s)	SEC. SFA	SOURCE(s)	!D'd	ISSUE(s)	SMM CODES
10/11/97	Strength		IR 97.12 (O7.2)	NRC	Outage Planning and Control - Planned outage work completed without difficulty. Planning and control continues to be a strength. 35 day outage work completed in 32-33 days.	1 2 3 4 5 A 0000 B 0000
10/11/97	Positive		IR 97-12 (O7.1)	NRC	Restart Affirmation by Plant Nurclear Safety Committee - Evaluation was thorough and effective.	1 2 3 4 5 A 0 0 0 0 0 B 0 0 0 0 0 C 0 0 0 0
10/11/97	Negative		IR 97-12 (O4.4)	LICENSEE	DG Autostart Defeated - URI - During maintenance on an electrical bus a DG autostart was defeated as part of the clearance without Ops recognition. URI closed 12/27/97.	1 2 3 4 5 A 00000 B 00000
10/10/97	Negative		IR 97-12 (O4.1)	LICENSEE	Inadvertent DG Start - Operator error while restoring system lineup. Lack of understanding and procedure step not clear.	1 2 3 4 5 A 00000 B 00000 C 0 000
MAINTEN	ANCE					
8/11/98	VIO		IR 98-07 (M4.1)	NRC	During the Spring 1998 Unit 1 outage an increase in errors was noted in maintenance and test activities. Several examples of multiple barrier breakdowns, such as independent verification, were identified as a violation of plant procedures. The licensee initiated a human performance improvement initiative to address the declining trend. See 6/6/98 PIM entry.	1 2 3 4 5 A 0 0 0 0 B 0 0 0 0 C 0 0 0 0
6/6/98	URI		IR 98-06 (M4.1)	NRC	The cross-connection of multiple phases of a breaker compartment resulted in a brief "fireball". No adjacent personnel or equipment were damaged and reactor make-up and control was maintained at all times. An Unresolved Item was identified for further review of this item and other related maintenance and test configuration issues.	1 2 3 4 5 A 0 0 0 0 B 0 0 0 0 C 0 0 0 0

-			-		BRUNSWICK		25-Nov-98
DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
5/8/98	Strength		IR 98-06 (M7.1)	NRC	The effectiveness of licensee controls over their ISI vendor and in identifying, resolving, and preventing problems were determined to be excellent. This was demonstrated by integrity of the vendor oversight observed; the conduct of a thorough assessment (B-OM-98-01) to determine the affectivenss of preoutage preparations in support of the outage; and the effective investigation documented in CR 98-0424 of discrepancies in inspection coverage and defect length sizing on Unit 1 & 2 core shroud welds.	A B C	
5/8/98	Strength		IR 98-06 (M2.2)	NRC	The material condition of the Unit 1 torus was very good in that, even below the water line, the paint was in excellent condition. Small isolated areas of discrepant paint identified by the inspector which needed attention were found to have already been identified by the licensee in the paint analysis for the torus. The completed weld repairs observed and the applicable documentation was found to be in accordance with procedures and specifications.	A B C	
5/8/98	Strength		IR 98-06 (M2.1)	NRC	Inservice Inspection activities observed were conducted in an excellent manner by examiners and analysts who were very knowledgable and skillful in their use of the examination methods. Enhanced inspection techniques were found to be very effective in thoroughly examining and evaluating the quality of welds.	A B C	
4/28/98	Weakness		IR 98-06 (M2.3)	NRC	For the most part, Foreign Material Exclusion controls were implemented satisfactorily. However, a lack of control of material in the fuel pool led to a thermocouple wire being inadvertently placed on top of the reactor vessel core. Also, spent fuel pool inventory and control was considered a weakness.	A B C	
4/16/98	NCV		IR98-06 (M8.4); LER 98- 01-00	LICENSEE	Reactor Building Roof Vent. Noble Gas Activity Monitor set at a value above allowed by TS. Procedure deficiency identified by licensee.	A B C	
3/31/98	VIO		IR 98-05 (M3.1)	NRC	Quality Control (QC) procedure step signed by non-qualified QC inspector. There were inconsistent requirements in a mechanical maintenance surveillance for Diesel Generator inspections. Nuclear Assessment had not reviewed a procedure revision that deleted QC steps.	A I B I C I	
3/31/98	Positive		IR 98-05 (M1.1)	NRC	Observed Maintenance activities conducted by the Fix It Now Team were conducted satisfactory and met the licensee's procedural requirements.	A [ B [ C [	

			-		BRUNSWICK		25-N	lov-93	1
DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SM	MC	ODES
3/26/98	VIO	PLT SUP	IR 98-05 (M3.2)	NRC	During the performance of the Transformer Deluge System Functional Test procedure steps were performed out of sequence.		1 2	3	4 5
						Α			
						В			
3/26/08	NCV		1D 00 04 (841 + 841 D)	NDC		С			
GIEGIOG	1101		IN 90-04 (MILL, MILZ)	NHC	Non-Cited Violation for failure to perform response time testing by TS and inadequate 50.59 review deleting requirements. Four apparent violations changed to		1 2	3 4	4 5
					one NCV.	A			
						в			
2/22/22						С			
3/20/98	Strength		IR 98-02 (M7.1)	NRC	Self-assessments of the Maintenance Rule were considered to be excellent and successfully monitored the effective implementation of the Maintenance Rule. The		1 2	3 4	4 5
					team considered the assessments performed to be a program strength.	A			
						в			
						С			
3/20/98	Strength		IR 98-02 (M2.1)	NRC	In general, plant material condition and housekeeping observed during walkdowns was excellent. However, the team did identify some minor discrepancies. Overall,		1 2	3 4	5
					the excellent material condition and housekeeping was considered a strength.	Α			
						В			
2/20/00	Danitiva		10 00 00 011 71			С			
3/20/96	POSITIVE		IH 98-02 (M1.7)	NRC	In general, for (a)(2) SSCs, detailed performance criteria had been properly established; appropriate trending had been performed; corrective actions were taken when SSCs failed to meet performance criteria or experienced failures; industry-wide operating experience had been considered, where practical; and operating data had		1 2	3 4	5
						A			
					been properly captured.	В			
2/20/200	Manhana					С			
3/20/98	vveakness		IH 98-02 (M1.6, M1.7)	NAC	A weakness in the Maintenance Rule process implementation was identified relative to: 1) proper alignment of system and component functions with performance		1 2	3 4	5
					monitoring groups (PMG), 2) omission of a corrective action p'an for a PMG, 3) use of incorrect equipment type codes in reviews for repetitive functional failures that	A			
					could lead to not identifying repetitive functional failures across system boundaries,	в			
					were made to unavailability start and stop times.	C			
3/20/98	Positive		IR 98-02 (M1.6)	NRC	In general, operating experience was being properly captured, and industry-wide operating experience was considered as approxiate		1 2	3 4	5
					operating experience was considered, as approriate.	A			
					8 1				
						c I			

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
3/20/98	Positive	terina de artes de Nora de Alta Maxima	IR 98-02 (M1.6)	NRC	Corrective actions, goals, and monitoring were comprehensive and were generally appropriate for the (a)(1) SSCs reviewed.		1 2 3 4 5
						A	
						в	
						С	
3/20/98	Positive		IR 98-02 (M1.6)	NRC	The licensee considered safety in establishment of goals and monitoring for the (a)(1) system and components reviewed		1 2 3 4 5
					and of the components reviewed.	A	00000
						в	
						С	0 000
3/20/98	Positive		IR 98-02 (M1.4)	NRC	The approach to balancing reliability and unavailability was reasonable. The detailed		1 2 3 4 5
					evaluation of balancing in the (a)(3) periodic assessment was considered good.	Δ	00000
						в	
						С	0 000
3/20/98	Positive		IR 98-02 (M1.3)	NRC	The (a)(3) periodic assessment was considered an excellent evaluation of the		1 2 3 4 5
					NUMARC 93-01 and paragraph (a)(3) of 10 CFR 50.65. The assessment met the	۵	00000
					requirements of paragraph (a)(3) of 10 CFR 50.65 and was considered to be a strength.	R	
				C	0 000		
3/20/98	Positive		IR 98-02 (M1.1)	NRC	Required structures, systems, and components (SSCs) were included within the		1 2 3 4 5
					scope of the Maintenance Rule.	٨	00000
						B	
						C	
3/20/98	Strength		IR 98-02 (Exec. Sum.)	NRC	Overall, the inspection team concluded that the licensee had a comprehensive		1 2 3 4 5
					Maintenenace Hule program that met the requirements of 10 CFR 50.65, and the program was being effectively implemented. The overall program was detailed, well		
					documented, and was considered to be a strength.	R	
						C	
3/4/98	Positive		IR 98-03 (M1.1, M1.4)	NRC	Maintenance activities observed were conducted in an effective and thorough		1 2 3 4 5
					manner by knowledgeable and skilled technicians. Procedures and drawings used had been verified as current. Satisfactory supervisory oversight was present.		
					had been vermed as current. Sausiactory supervisory oversight was present.	P	
						C	
						-	

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SI	MM	CODE	s
2/12/98	Negative	PLT SUP	IR 98-05 (M7.1)	LICENSEE	There was a long standing history of fire door hardware deficiences. Various licensee assessments identified these deficiencies and these represented missed opportunities to correct the adverse conditions.	AB			4 5	
2/12/98	VIO	PLT SUP	IR 98-05 (M4.2)	NRC	Failure to report a condition outside the 10 CFR 50 Appendix R design basis concerning unreviewed modifications to fire doors between reduntant safety equipment. (7/1/98 Licensee Contested)	C				
2/12/98	VIO	PLT SUP	IR 98-05 (M4.1), IR 97-15 (F4.1)	LICENSEE	Unreviewed modifications were made to 28 Diese! Generator Building fire doors which introduced a common mode failure. The modification degraded the three hour fire barrier separating redundant safety equipment. (7/1/98 Licensee Contested)	C				
2/5/98	Negative		IR 98-03 (M1.5)	NRC	The programs governing control of component leakage and the maintenance backlog were ineffective in the timely resolution of a degrading secondary containment penetration link seal. The failure to promptly correct this situation has resulted in a spread of contamination into clean areas and the establishment of long- term contaminated areas.	C				
2/3/98	Negative		IR 98-03 (M1.7)	NRC	Red-line changes made to vendor procedures were consistent with the requirements of licensee procedures and Technical Specifications. Improvements were needed in monitoring training certification of contractor qualification.	C				
1/12/98	Positive		IR 97-15 (M1.1)	NRC	Routine Maintenance Activities - Licensee continuing to upgrade the material condition of plant equipment and equipment spaces.	B C A				
1/9/98	NCV		IR 97-15 (M2.2)	LICENSEE	Control of Special Processes - Inadequate control of shroud weld examination. Comparison UT data indicate H6B weld crack - data inconsistent.	B C A B C				

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
1/7/98	Weakness		IR 97-15 (M2.1)	NRC	Surveillance Test - Tests performed satisfactorily but weakness is foreign material exclusion for test equipment.	A B	
1/4/98	VIO		IR 97-15 (M4.1)	NRC	Corrective Action Violation - Process computer failures not documented on CR needed for Maint. Rule Input Data.	C A B	
12/11/97	VIO		IR 97-13 (M1.3)	NRC	EQ Maintenance - PM procedure did not indicate inspection requirements of seals in Motor Control Centers.	ABC	
12/11/97	Positive		IR 97-13 (M1.3)	NRC	EQ Maintenance - EQ maintenance of electrical equipment was conducted in a thorough and effective manner.	A B C	
12/2/97	VIO		IR 97-13 (M3.1)	NRC	Abnormal Values not Detected in Daily Surveillance Log - Abnormal values for Steam Jet Air Ejector radiation monitor not red-circled on logs.	A B C	
12/2/97	Positive		IR 97-13 (M1.1)	NRC	Fuel Shipping Cask Movement - Cask moved in accordance with approved methodology with good supervisory oversight.	A B C	
11/5/97	Positive		IR 97-12 (M1.3)	LICENSEE	Surveillance Testing - Testing stopped once a discrepancy was identified. Action taken to correct procedure and drawing.	A B C	

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
10/11/97	Positive		IR 97-12 (M6.1)	NRC	Measuring and Test Equipment - M&TE found properly lab	eled and calibrated. A B	
10/3/97	LER		LER 2-97-04	LICENSEE	Safety Relief Valves Exceeded TS Setpoint Limits - Five of pressures outside TS limits. Power Uprate change tolerand 3. Only one SRV was -3.2.	C 11 SRVs found to lift at ce from + or - 1 to + or - A B C	
10/3/97	LER		LER 1-97-13	LICENSEE	RCIC Surv. Procedure Not Adequate - Found during GL 96 verify ability of RCIC exhaust diaphragm high pressure char	i-01 review. Test did not nnel logic. A B C	
10/2/97	Negative		IR 97-12 (M2.3)	LICENSEE	Shroud Crack Inspections - Examinations showed little crac HGB gave conflicting information, depth less than perviousl assist. (See NCV 1/9/98)	ck growth, however weld y measured, EPRI to A B C	
10/2/97	Positive		IR 97-12 (M2.2)	NRC	In-vessel Inspections - Inspections performed by skilled teo examinations good.	hnicians and A B C	
10/2/97	VIO		IR 97-12 (M1.3)	LICENSEE	Clearance Errors - Incorrect breaker racked out, cooling wa isolated while plant in hot standby caused temp. limit of sea	ater to recirc. pump seals to be exceeded. A B C	
ENGINEE	RING						
6/30/98	Negative		IR 98-07 (E3,1)	NRC	The inspector found that the licensee had maintained Specie (SNM) accountability for a Local Power Range Monitor (LPP inspector found that the licensee's warehouse inventory had of LPPM units on hand due to not having well defined proce in their material control procedures.	al Nuclear Material IM) in question. The an error in the number dural guidance on SNM B C	
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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
6/24/98	AIO		IR98-07 (E2.1)	NRC	The inspector identified a corrective action violation in the identification and untimely initiation of corrective actions regarding an adverse condition involving the Run Control Relay and Jet Assist Time Relay DG air bellows time delay relays.	AB	
6/24/98	MISC		IR 98-07 (M2.1)	LICENSEE	The licensee identified a 63-inch crack in a fuel rod from the Unit 1 core. Debris was determined to be the initiator of the failure.	C A B	
6/11/98	Negative		IR 98-07 (E7.1)	NRC	The licensee acknowledged that design control activites warranted additional attention. These activities were accuracy of Equipment Database System, staffing of design control organization, and UFSAR updates.	C A B	
6/6/98	Strength		IR 98-06 (E2.3)	NRC	The inspector concluded that the same excellent planning and decision-making which was implemented during the Unit 2 Emergency Core Cooling System (ECCS) strainer modifications led to the successful completion of the Unit 1 ECCS strainer modification.	C A B	
5/22/98	Positive		IR 98-06 (E2.2)	NRC	The licensee's operability evaluation of bolted connections for RPV pipe support was completed in accordance with the guidance specified in Generic Letter 91-18.	C A B	
5/22/98	Positive		IR 98-06 (E2.1)	NRC	The licensee's actions to repair the pinhole leaks in the Unit 1 SLC flexible piping were performed in accordance with good engineering practices and NRC requirements.	A B C	
5/22/98	Negative		IR 98-06 (E1.3)	NRC	The licensee was making adequate progress in resolving and closing CRs identified by the EQ group. However, engineering failed to recognize that a repair should be made to the Okonite T95 tape splices on the drain wires for two Containment Atmosphere RTDs to prevent multiple grounds in the circuit. This was considered a weakness.	A B C	

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		S	MM	COD	DES
5/21/98	VIO		IR 98-06 (E4.1)	NRC	A change to the Diesel Generator load test was made which altered the test loading profile. The use of a temporary change to alter the intent of the test load procedure was identified by the inspector as a violation.	A B			4 ] [] ] [2]	5
5/19/98	Negative		LER 98-03-00	LICENSEE	Safety Relief Valves Exceeded TS Setpoint Limits - modification to fix did not work - suppl. LER expected	C A B				5
4/11/98	Positive		iR 98-05 (E2.1)	NRC	The licensee provided effective oversight for the design and installation of the Control Building Air-Conditioning upgrade modification.	C				5
3/26/98	Negative		IR 98-05 (£3.1)	SELF	Due to the inability of the Standby Liquid Control system piping heat trace to maintain adequate temperatures to maintain the chemicals in solution during cold temperatures the licensee implemented more limiting administrative limits than existed in Technical Specifications. A modification was planned for the summer of 1998.	C A B				5
3/20/98	Strength		IR 96-02 (E4,1)	NRC	System engineers were very knowledgeable of their systems, were proactive in corrective actions, and had a good understanding of Maintenance Rule requirements and how to apply the Rule to their system.	C				5
3/20/98	Positive		IR 98-02 (M1.5)	NRC	The use of the equipment out-of-service (EOOS) computer program to evaluate plant configurations was also good, as was the process for ensuring that critical safety functions were available during planned outages.	BC				5
3/20/98	Positive		IR 98-02 (M1.5)	NRC	The overall approach, under paragraph (a)(3) of the Maintenance Rule, to assessing the risk-impact of maintenance activities was good.	B C A B C				

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID,q	ISSUE(s)		SMM	CODES
3/20/98	Positive		IR 98-02 (M1.2)	NRC	Based on the review of the sample SSCs, the licensee's approach to risk-ranking for the Maintenance Rule was good.		1 2 3	4 5
						Α	000	
						В		
3/20/98	Strength		ID 00 00 (MI 0)	NDC	The second se	С		
	Guongar		111 30-02 (M11.2)	NHC	The expert panel committee meeting discussions on covered topics were excellent. The expert panel meeting minutes were well documented, and were considered a		1 2 3	4 5
					strength.	Α		
						в		
2/20/20						С		
3120136	Positive		IR 98-02 (M1.2)	NRC	The current method of assuring the assumptions for reliability and availability in the PSA are conserved was good.		1 2 3	4 5
						A	000	
						в		
						С		
3/20/98	Positive		IR 98-02 (M1.2)	NRC	The overall quantitative approach used to perform risk ranking for SSCs in the scope of the Maintenance Bule was good. Performance criteria were established with		1 2 3	4 5
					substantial plant safety analysis (PSA). Documentation of PSA input was good.	A	000	
						в		
						С		
3/14/98	NCV		IR 98-03 (E8.3)	LICENSEE	A non-cited violation was identified for failing to perform a 10 CFR 50.59 review when downgrading the quality classification of the Control Building air conditioning		1 2 3	4 5
					units.	A	000	
						в		00
						С		
3/12/98	VIO		IR 98-04 (E8.3)	NRC	Violation for using non-design verified engineering evaluation to change plant design		1 2 3	4 5
					documenta.	A	000	
						в		
						С		
3/6/98	Positive		IR 98-03 (E1.1)	NRC	Based on completed and planned licensee actions, the inspectors concluded that		1 2 3	4 5
					and had met the intent of GL 89-10, Motor-Operated Valve program.	A		
						B		
					and had the intent of GL 83-10, Motor-Operated Valve program.	B		

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
2/17/98	Negative		IR 98-03 (E2.1)	NRC	Operator training for the oncoming crew on the compensatory measures during upgrades of the Control Building Ventilation System was not timely.		1 2 3 4 5
						Α	00000
						В	
1/4/98	Negative		IR 97.15 (E2 1)	NDC	Improved Standard TC Implementation Income to 1170 to 1170	С	
			11 07-10 (CE.1)	NHC	instructions and changes indicated additional clarification and licensee review were		1 2 3 4 5
					needed.	Α	00000
						В	
12/12/97	Positive		IB97-13 (E1.2) NRC FO Program Reconstitutation Programs - Programs being mode in closure -		С		
	1 USAIVE		(E1.2)	NHC	and CRs Training meets NRC requirements. Inst. setpoint calcs. adequate.		1 2 3 4 5
						Α	00000
						В	
12/12/97	Manhanan		1007 10 (51.1)			С	
12/12/37	treatness		IH97-13 (E1.1)	NHC	EQ Procedure Reference Wrong - Reference to a drawing for accident temperature data was not available for use and inconsistencies in procedure.		1 2 3 4 5
						A	00000
						В	
10/10/07	100					С	
12/12/97	00		IH 97-13 (E1.1 & E1.4)	NRC	Inadequate EQ Maintenance Procedure - Difference greases for EQ electric motor specified in procedure had not been evaluated by EQ program.		1 2 3 4 5
						Α	00000
						в	
10001000						С	
10/31/97	ED		IR 97-12 (E8.1)	LICENSEE	Failure to Perform 50.59 Review for Spent Fuel Shipping Cask EA 97-521 - Enforcement Discretion granted 12/10/97.		1 2 3 4 5
						A	00000
						в	
						С	
10/31/97	Negative		IR 97-12 (E7.1)	LICENSEE	NAS Audit of Engineering - NAS audit adequate but showed corrective actions have been ineffective		1 2 3 4 5
						A	00000
						в	00000
						С	

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
10/31/97	Positive		IR 97-12 (E1.2)	NRC	Modifications for USI A-46 - Mods. adequately implemented in accordance with design requirements.	AB	
10/31/97	VIO		IR 97-12 (E1.1)	NRC	EQ Program Lack of Progress - 2 VIOs - Failure to take corrective action to correct EQ problem and failure to document operability of equipment.	C A B	
10/13/97	VIO		IR 97-12 (E3.1)	LICENSEE	Errors in Database for Minimum Critical Power Ratio Safety Limit Calculation - Licensee failed to correct database after a problem in April occurred. Another error found September 1997.	ABC	
10/13/97	Licensing		IR 97-12 (E2.3) LER 1-97- 11	NRC	Torus Bypass Flow - Enf. conf. 12/19/97. No violation, licensee reported before Part 21 issued.	A B C	
10/11/97	Strength		IR 97-12 (E2.1)	NRC	Torus Suction Strainers Modification - Significant strengths in project management. Excellent planning and decision making led to the successful completion of a major plant modification.	A B C	
7/27/98	Positive		IR 98-07 (F7.1)	LICENSEE	The triennial fire protection audit was determined to be a thorough review of the program. Audit findings reflected extensive auditor knowledge of fire protection and the associated requirements. Additional assessments were scheduled to independently review corrective actions for the fire protection program.	AB	
7/17/98	Positive		IR 98-07 (R1.1)	NRC	The licensee's radiation control practices for radiological posting and labeling, personnel dosimetry, radiological surveys, RCA ingress and egress control, and whole body counting were consistent with the licensee's Radiation Control and Protection Manual and relevant sections of 10 CFR 20.	A B C	
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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)	SMM CODES
5/29/98	VIO		IR 98-06 (R1.3)	NRC	The licensee failed to take prompt action when a continuous air monitor and an additional sample indicated abnormal airborne activity. As a result, five workers were present in an area of airborne radioactivity levels greater than 0.25 DAC (Derived Air Concentration), without appropriate radiation work permits. A violation was issued for the failure to properly implement and establish procedures that outlined those actions to be taken in the event of survey results indicating abnormal airborne activity.	1 2 3 4 5 A 0000 B 0000 C 0 000
5/22/98	VIO		IR 98-06 (R1.1)	NRC	An unresolved item was identified to further evaluate the adequacy of detection methods for detecting Fe-55 concentrations or quantities on material free released from the site.	12345 A 0 0 0 0 0 B 0 0 0 0 0 C 0 0 0 0
5/22/98	URI		IR 98-06 (R1.1)	NRC	A violation was identified for failure to perform nine procedurally required routine radiation surveys.	1 2 3 4 5 A 00000 B 00000 C 0 000
5/22/98	Strength		IR 98-06 (R1.1)	NRC	Contamination controls were effective in containing high levels of contamination to appropriate areas.	1 2 3 4 5 A 0 0 0 0 B 0 0 0 0 C 0 0 0 0
5/14/98	Positive		IR 98-06 (S1.2)	NRC	The inspector determined, through Access Authorization procedures and records review, that the licensee had established adequate procedures as required by 10 CFR 73.56 for the review of a denial or revocation by the licensee of unescorted access authorization, at the request of an affected employee.	1 2 3 4 5 A 0 0 0 0 0 B 0 0 0 0 0 C 0 Ø 0 0
4/25/98	Positive		IR 98-05 (R8.1)	NRC	The root cause analysis for a cited violation concerning chemical control was excellent. Subsequent self-assessments were self-critical and effective in the identification of issues.	1 2 3 4 5 A 0 0 0 0 0 B 0 0 0 0 0 C 0 0 0 0
4/22/98	Strength		IR 98-06 (R1.1)	NRC	Radioactive material was labeled in accordance with requirements and appropriately stored. Radiation and high radiation areas were properly posted and controlled.	1 2 3 4 5 A 0 0 0 0 0 B 0 0 0 0 0 C 0 0 0 0

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM	CODES
4/20/98	VIO		IR 98-06 (F3.1)	NRC	The procedures for maintaining minimum staffing for Alternate Safe Shutdown (ASSD) during a fire were determined to be inconsistent with 10 CFR 50 Appendix R. 10 CFR 50 Appendix R requires that all personnel needed for ASSD be onsite at all times. However, the procedures for ASSD staffing allowed less than minimum staffing for 14 days. A violation was issued for the failure to properly establish fire protection procedures in accordance with 10 CFR 50.	A B		4 5
4/20/98	VIO		IR 98-06 (F1.1)	NRC	Staffing for alternative safe shutdown procedures was not implemented in accordance with procedures. The failure to maintain adequate Alternate Safe Shutdown staffing was identified as a violation. A violation was also identified for the failure to retain QA documents requested during this inspection.	C A B		
4/3/98	Positive		IR 98-05 (S1.1)	NRC	Security activitives during observation in the CAS and SAS were satisfactory. Lighting inside the protected area was acceptable with one exception that was corrected.	C A B		
3/14/98	Positive		IR 98-03 (R1.3)	NRC	The licensee effectively implemented training and qualification for Radiation Protection and Chemistry Technicians in accordance with the training program procedural requirements.	C A B		
3/14/98	Positive		IR 98-03 (R1.2)	NRC	The licensee was generally effective at identifying and correcting radiological control related deficiencies in accordance with the corrective action management program.	A		
3/14/98	Positive		IP. 98-03 (R1.1)	NRC	Radiation control practices for Radiation Control Area ingress and egress control, personnel dosimetry, radiological surveys and postings, and container labeling were consistent with the licensee's Radiation Control and Protection Manual and relevant sections of 10 CFR 20.	A		
3/12/98	Positive		IR 98-201 ( Cover Ltr)	NRC	Very positive ORSE team inspection. Significant involvement by Operations, protective strategy effective and demonstrated excellent capabilities against threat. Effective measures for safeguards measures.	A [ B [ C [		

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
3/10/98	Negative		IR 98-05 (R4.1)	SELF	Work activities on the refuel bridge grapple that resulted in skin contaminations were covered by the radiological work permit and work instructions. However, work practices to prevent the spread of contamination were not effective.	A B C	
3/8/98	VIO		IR 98-05 (F3.1)	NRC	The engine driven fire pump flow test procedure did not demonstrate that the pump was operable until the procedure was revised and acceptance criteria changed. This was similar to an event in June 1997 where a dual unit shutdown was commenced as a result of erroneous data from an inadequate test procedure. NRC concern noted over the proper establishment and implementation of the fire protection program.	ABC	
1/29/98	MISC		IR 98-01 (P4.2)	NRC	Except for the exercise weakness, command and control of the Emergency Operations Facility (EOF) and timeliness of off-site notifications were good. Off-site dose projections were completed satisfactorily, although a software inconsistency was detected but not fully evaluated during the exercise. Restrictions on food consumption were not effectively communicated to all EOF staff. Status boards were well maintained, but position logs were not. The State of North Carolina was effectively integrated into the EOF organization.	A B C	
1/29/98	Negative		IR 98-01 (P4.2)	LICENSEE	The Emergency Operations Facility failed to fulfill a primary facility responsibility by not providing off-site agencies with off-site Protective Action Recommendations in a timely manner. This failure was identified as an exercise weakness.	A B C	
1/29/98	Positive		IR 98-01 (P4.2)	NRC	Command and control of Technical Support Center operations by the Site Emergency Coordinator were judged to be exemplary. Emergency declarations were made timely, and accident mitigation activities were well coordinated with the Operational Support Center.	A B C	
1/29/98	Positive		IR 98-01 (P4.2)	NRC	The Shift Superintendent/Site Emergency Coordinator and the Control Room Simulator staff were very effective in performing initial emergency responsibilities.	A B C	
1/29/98	Positive		IR 98-01 (P4.1)	NRC	The scenario developed for this exercise was effective for testing the integrated emergency response capability, and exercise preparations were organized.	A B C	

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DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)		SMM CODES
1/2/98	Negative		IR 97-15 (R4.1)	NRC	Health Physics Technician Walkthrough - Procedure lacked guidance to ensure continuity of walkthroughs.	AB	
12/27/97	Negative		IR 97-13 (R1.2)	NRC	Peer Radiological Work Practices - Inspector found several poor work practices in a radioactive material storage area.	C A B	
12/12/97	Positive		IR 97-13 (F7.1)	NRC	NAS Assessments of Fire Protection - Effective in identifying issues. Audits were substantial.	C A B C	
12/12/97	Positive		IR 97-13 (F6.1)	NRC	Fire Brigade Organization - Licensee got rid of Loss Prevention Unit and re- assigned fire protection to Operations. This change met NRC requirements.	ABC	
12/12/97	Positive		IR 97-13 (F5.1)	NRC	Fire Brigade Organization - Org. and Training met site procedures. Equipment properly maintained.	A B C	
12/12/97	URI		IR 97-13 (F2.2) (F3.1)	NRC	UFSAR Discrepancy - Documentation of Fire Door and Frame Evaluations not in Fire Hazard Analysis section UFSAR.	A B C	
12/12/97	Negative		IR 97-13 (F2.1)	NRC	Fire Seal Testing Documentation - Design adequate but records not available.	A B C	

					25-Nov-98			
DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID.q	ISSUE(s)		SMM COL	DES
12/12/97	VIO		IR 97-14 (R1.1)	NRC	Failure to write a condition report - Licensee did not write a condition report upon identifiying a Locked High Radiation Area as not controlled.	A B		5
12/12/97	VIO		IR 97-14 (R 1.1)	NRC	Control of Locked High Radiation Area - Failure to control a locked high radiation area.	C A B		
12/12/97	Positive		IR 97-14 (R1.1)	NRC	HP Inspection - Radiation Control Program effectively implemented with good occupational exposure controls observed during normal plant operations.	C A B		5
12/11/97	Positive		IR 97-13 (F1.1)	NRC	Fire Protection System Corrective Maintenance - Maintenance was timely and material conditions satisfactory.	C A B C		5
10/13/97	Positive		IR 97-12 (R1.1)	NRC	Radiation Control Practices During Outage - Improved supervisory oversight was observed on refuel floor, drywell, and torus.	ABC		5
10/3/97	VIO		IR 97-12 (S4.1)	NRC	Inadequate Security Procedure Procedure did not direct actions to lock turnstile for unauthorized entry. Licensee denied but NRC concluded still VIO.	A B C		5

					BRUNSWICK			25-Nov-98
TE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)			SMM CODE
SMA	Template Co	des:		SALP	Functional Areas:	ID Code:		
1B 1C 2A 2B 3A 3B 3C 4A 4B 4C 5A 5B 5C	OPERATION PER OPERATION PER OPERATION PER MATERIAL COND MATERIAL COND HUMAN PERFOR HUMAN PERFOR HUMAN PERFOR ENGINEERING/DI ENGINEERING/DI ENGINEERING/DI PROBLEM IDENT PROBLEM IDENT	FORMANCE - No FORMANCE - O FORMANCE - Pr ITION - Equipmer ITION - Program MANCE - Work Pr MANCE - KSA MANCE - Work En ESIGN - Design ESIGN - Design ESIGN - Engineeri ESIGN - Program IFICATION & SOL IFICATION & SOL	Irmal Operations perations During Transients ograms and Processes at Condition s and Processes atformance vironment ing Support is and Processes UTION - Identification UTION - Analysis UTION - Resolution	ENG MAINT OPS PLT SU SAQV	ENGINEERING MAINTENANCE OPERATIONS PLANT SUPPORT SAFETY ASSESSMENT & QV	LICENSEE NRC SELF	LICENSEE NRC SELF-REVEALED	•

EEIs are apparent violations of NRC requirements that are being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action" (Enforcement Policy), NUREG-1600. However, the NRC has not reached its final enforcement decision on the issues identified by the EEIs and the PIM entries may be modified when the final decisions are made. Before the NRC makes its enforcement decision, the licensee will be provided with an opportunity to either (1) respond to the apparent violation or (2) request a predecisional enforcement conference.

URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.

Last Updated: 8/11/98