

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

PORTLAND GENERAL ELECTRIC COMPANY

THE CITY OF EUGENE, OREGON

PACIFIC POWER AND LIGHT COMPANY

DOCKET NO. 50-344

TROJAN NUCLEAR PLANT

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 143 License No. NPF-1

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Portland General Electric Company, et al., (the licensee) dated February 17, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-1 is hereby amended to read as follows:
  - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.143, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Director

George W. Knighton, Director Project Directorate V Division of Reactor Projects - III, IV, V and Special Projects Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: June 16, 1988

# ATTACHMENT TO LICENSE AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NO. NPF-1 DOCKET NO. 50-344

Revise Appendix A as follows:

Remove Pages		Insert Pages		
3/4	3-55		3/4 3-55	
3/4	3-56		3/4 3-56	
3/4	3-58		3/4 3-58	

#### TABLE 3.3-12

## RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

INS	TRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
۱.	Gross Radioactivity Monitors Providing Automatic Termination of Release			
	<ul> <li>a. Liquid Radwaste Effluent Line (PRM-9)</li> </ul>	1	•	23
	b. Steam Generator Blowdown Effluent Line (PRM-10) or Steam Generator Blowdown Ion Exchanger Effluent (P2M-17)	1	• •	24
2.	Gross Radioactivity Monitors Not Providing Automatic Termination of Release			
	a. Component Cooling Water System Train A (PRM-7)	1	At all times	25
	<ul> <li>b. Component Cooling Water System Train B (PRM-8)</li> </ul>	n 1	At all times	25

\* During releases via this pathway.

## TABLE 3.3-12 (Continued)

## RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

INS	TRUP	<u>IENT</u>	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
3.	Flo	ow Rate Measurement Devices			
	a.	Liquid Radwaste Effluent Line (FT-4044 or FIY-4098)	1	•	26
	b.	Discharge Structure (FR-3495)	1	•	26
	c.	Steam Generator Blowdown Effluent Line (FI-6715 or FI-4921)	1	* .	26

\* During releases via this pathway.

#### TABLE 4.3-8

#### RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INS	STRUMENT	CHANNEL	SOURCE CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST
۱.	Gross Beta or Gamma Radioactivity Moniters Providing Alarm and Automatic Isolation				
	a. Liquid Radwaste Effluents Line (PRM-9)	D*	Р	R(1)	Q(1)
	b. Steam Generator Blowdown Effluent Line (PRM-10)	D*	M	R(1)	Q(1)
	c. Steam Generator Blowdown Ion Exchanger Effluent (PRM-17)	D*	H	R(1)	Q(1)
2.	Gross Beta or Gamma Radioactivity Monitors Providing Alarm But Not Providing Automatic Isolation				
	Component Cooling System (PRM-7, -8)	D	м	R(1)	Q(1)
3.	Flow Rate Monitors				
	a. Liquid Radwaste Effluent Line (FT-4044/FIY-4098)	D*	N/A	R**	N/A
	b. Discharge Structure (FR-3495)	D*	N/A	R**	N/A
	c. Steam Generator Blowdown Line (FI-6715 or FI-4921)	0*	N/A	R**	N/A

\* During releases via this pathway.

\*\* Applies only to flow indication loop.

- 1) Instrument indicates above alarm trip setpoint.
- 2) Instrument indicates a downscale failure.

3) Controls not in OPERATE mode, or instrument de-energized.

The Channel Functional Test shall, where applicable, include verification that automatic isolation of the effected pathway and/or control room annunciator occurs if: