

# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555 December 7, 1988

LICENSEE: Florida Power Corporation (FPC)

FACILITY: Crystal River Unit 3 (CR-3)

SUBJECT: CRYSTAL RIVER UNIT 3 - SUMMARY OF NOVEMBER 29, 1988 MEETING ON

ANTICIPATED TPANSIENTS WITHOUT SCRAM (ATWS)

On November 29, 1988, representatives of FPC met with representatives of the NRC staff to discuss open ATWS issues at CR-3 in an attempt to reach agreement on the conceptual design. The list of attendees, agenda, and slides discussed during the meeting are enclosed.

FPC presented a brief description of the Emergency Feedwater Initiation and Control (EFIC) system to clarify its functions and interfaces with ATWS. In response to a question, FPC stated that bypassing EFIC with the ATWS Mitigation System Actuation Circuitry (AMSAC) signal to initiate Emergency Feedwater (EFW) flow would violate NUREG-0737 requirements for class IE EFW initiation and control. Spurious AMSAC actuation could endanger EFW control, possibly resulting in serious overcooling events. In total, such a design would not represent an increase in overall plant safety.

The issue of independence of ATWS from the Reactor Trip Systems (RTS) was discussed only briefly since this aspect of the conceptual design was essentially resolved.

With regard to diversity, FPC stated that EFIC is separate and diverse from the Reactor Protection System (RPS). The ATWS logic (Bailey 820) was described as different from the RTS logic (Bailey 880). FPC agreed to submit additional schematics and descriptions to support this position as part of the final design submittal. For the conceptual design, presently submitted information is acceptable. FPC also noted that the reactor coolant pump input to AMSAC is picked up directly from the pump power monitor upstream of any EFIC logic. FPC will submit additional supporting information as part of the conceptual design.

Isolation of ATWS from other systems was said to be the same as the isolation used for the Safety Parameter Display System (SPDS) and Regulatory Guide 1.97 applications. As part of the conceptual design submittal, FPC will verify this, provide the model number and manufacturer of the isolators and state that their application in ATWS does not invalidate use of these isolators. Additional details will be presented with the final design package.

NRC representatives at the weeting stated that other than receipt of the information stated above, there were no apparent problems with the ATWS conceptual design as now documented.

NRC requested that FPC expedite submitta: of the final design package now scheduled for June 1989. FPC stated that this schedule represents full construction design information, which NRC stated was not required for our review. NRC agreed to

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ORIGINAL SIGNED BY

Harley Silver, Project Manager Project Directorate II-2 Division of Reactor Projects-I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Attendance List

2. Agenda

3. Slides

cc w/enclosures: See next page

[MEETING SUMMARY/CRYSTAL RIVER]

DWHTer 12/6/88 HSilver/jd 12/7/88

ATWS equipment by the next refueling.

NRR: 8018 SNEWberry 12/7 /88 D:P01//2 HRetkow 12/7 /88 Mr. W. S. Wilgus Florida Fower Corporation

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Director, Nuclear Site
Florida Power Corporation Support
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### ENCLOSURE 1

NRC-FPC MEETING ON ATWS FOR CR-3

November 29, 1988

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Harley Silver

Vincent Thomas

freat Collins

Jack Tunstill

Bert Simpson

Jerry L. Mauck

Bob Stevens

Richard Low

Ken Wilson

Carl C. Strempke

Scott Newberry

Stu Long

#### ORGANIZATION

NRR/PM

NRR/ICSB

INEL

FPC

FPC

NRR/ICSB

NRR/ICSB

FPC

FPC

Gilbert Assoc. Inc.

NRR/SICB

ACRS

#### AGENDA NOVEMBER 29, 1988

#### NRC/FPC ATWS MEETING

The purpose of this meeting is to resolve all open ATWS mitigation design issues. Both NRC and FPC have benefit of 11/18 RAI and 11/22 FPC submittal and should be able to clearly understand any remaining differences or misunderstandings. Outcome should be means to document sufficient final closure to support FPC design and installation by Refuel 7.

- I. Introduction
- II. EFIC Design Overview
- III. Discussion of Key Issues
  - A. Independence
  - B. Diversity
  - C. Isolation
  - D. EFIC Interface
- IV. Resolution of key issues
  - A. Closed by FPC 11/22 Submittal
  - B. Closed by FPC 11/22 Submittal and Final Design
  - C. Closed by FPC 11/22 Submittal and Final Design
  - D. Resolved at meeting
  - V. Discussion of Final Documentation of NRC Acceptance of Conceptual Design

#### **EFIC OVERVIEW**

# I. INTRODUCTION

# II. OVERVIEW OF CHANNELS

- a. channels A,B,C,&D ----slides 1,2,3,4,
  - 1. inputs
  - 2. initiate & trip outputs
  - 3. actuation logic

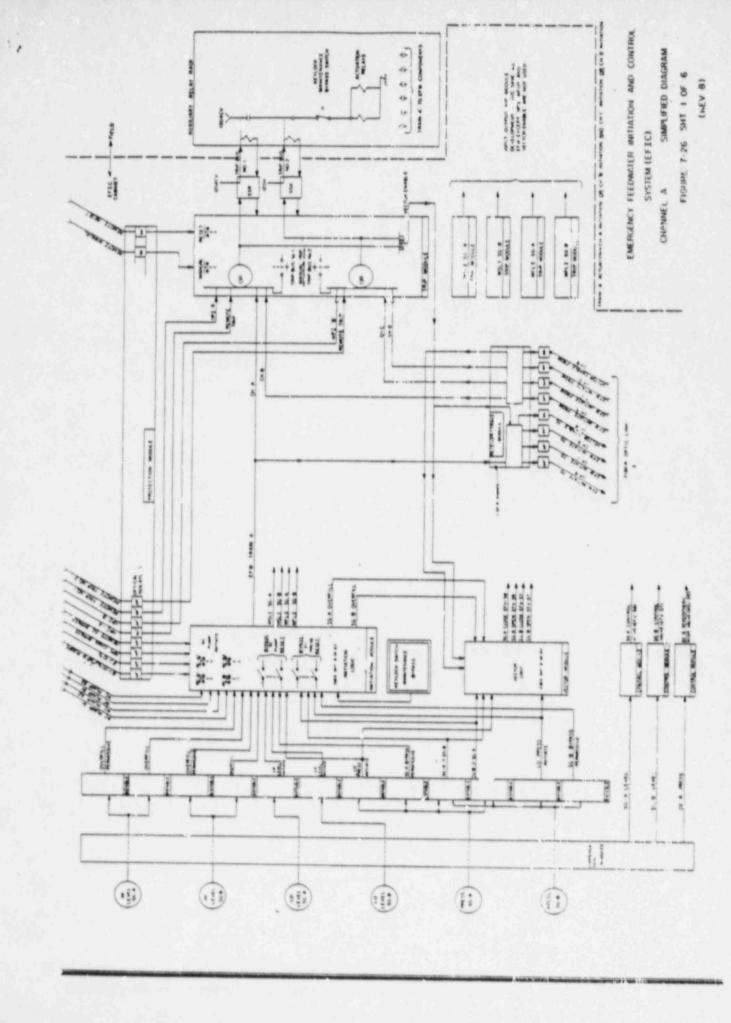
# III. INITIATION MODULE

- a. logic----slide 5
- b. schematic (partial)----slide 6
  - 1. RCP optic isolators
  - 2. FW inputs OR gates

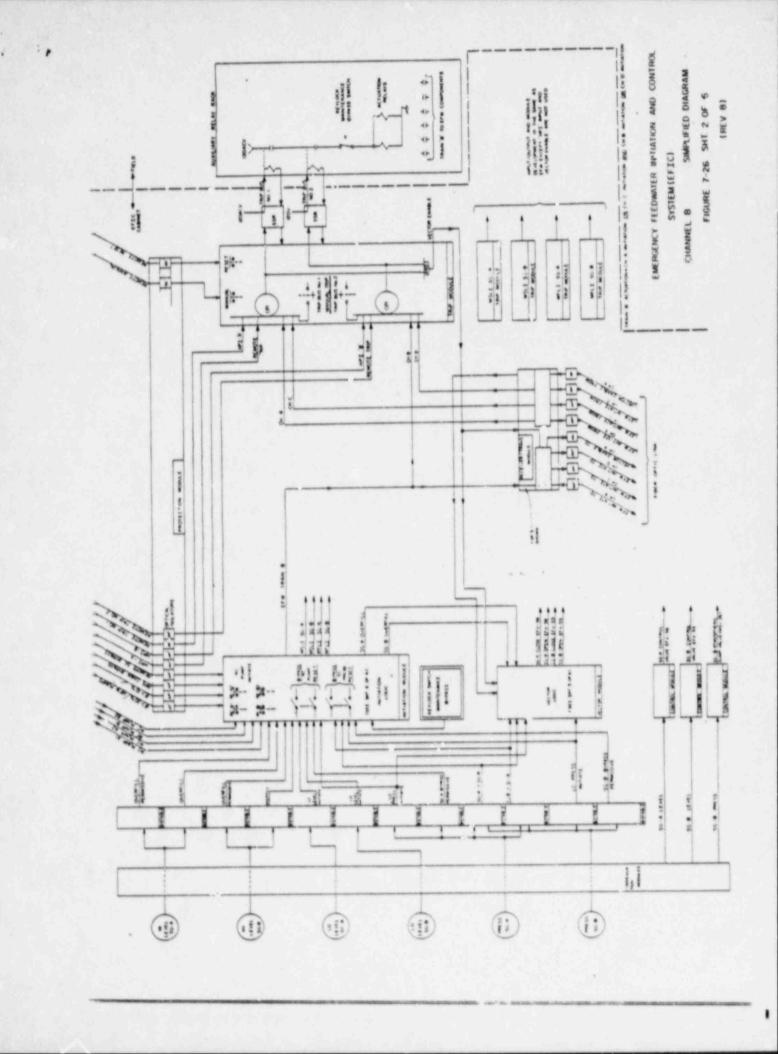
# IV. AMSAC INTERFACE

- a. A,B,C & D overview, GAI dwg -----slides 7,8,9,10
- b. FP Ty ----slide 11
- c protection rodule ----slide 12

# V. QUESTIONS

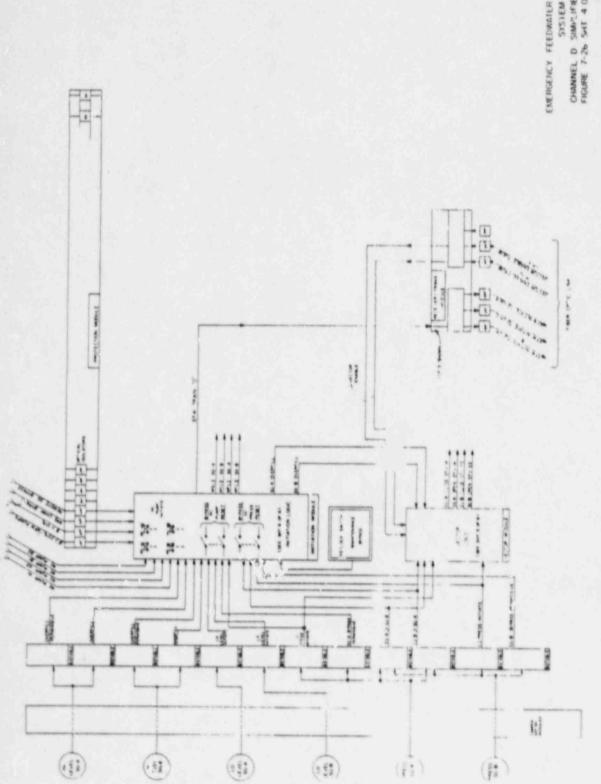


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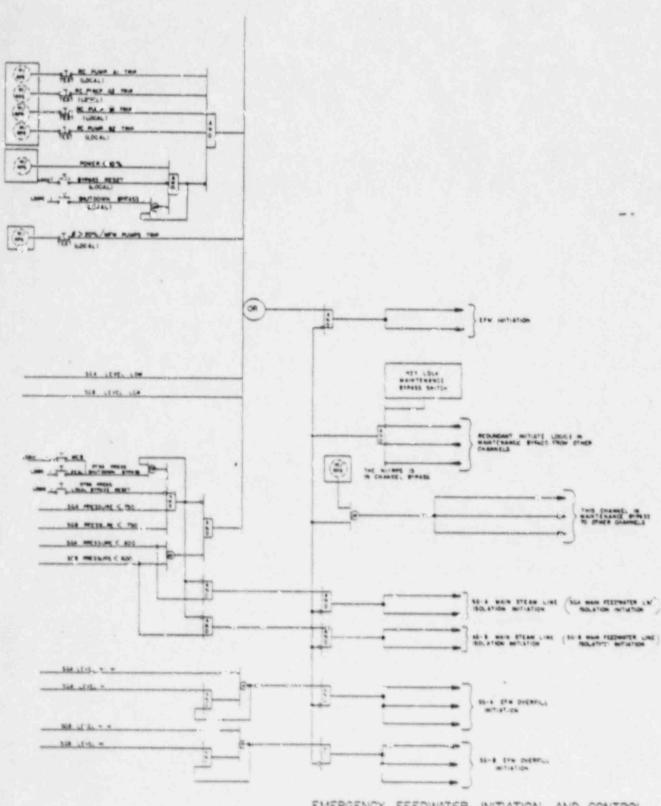


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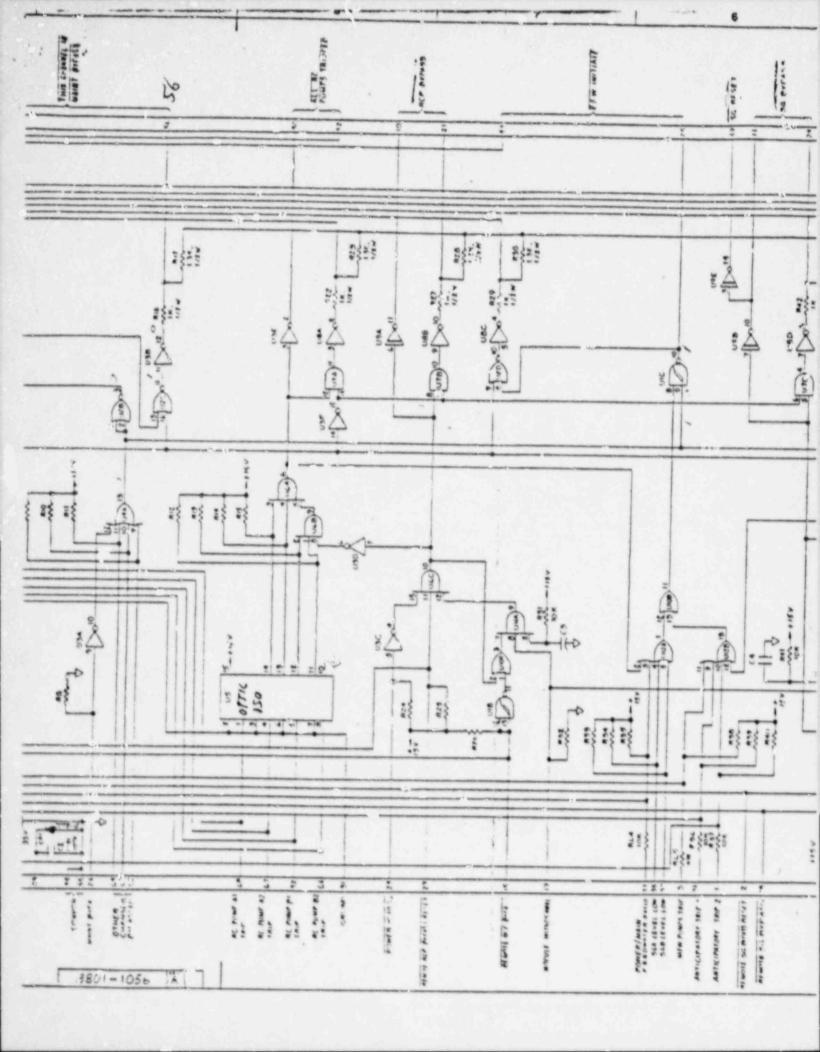
EMERGENCY FEEDWATER INITIATION AND CONTROL SYSTEM LEFIC)
CHANNEL C SIMPLIFIED DIAGRAM
FIGURE 7-26 SHT 3 OF 6

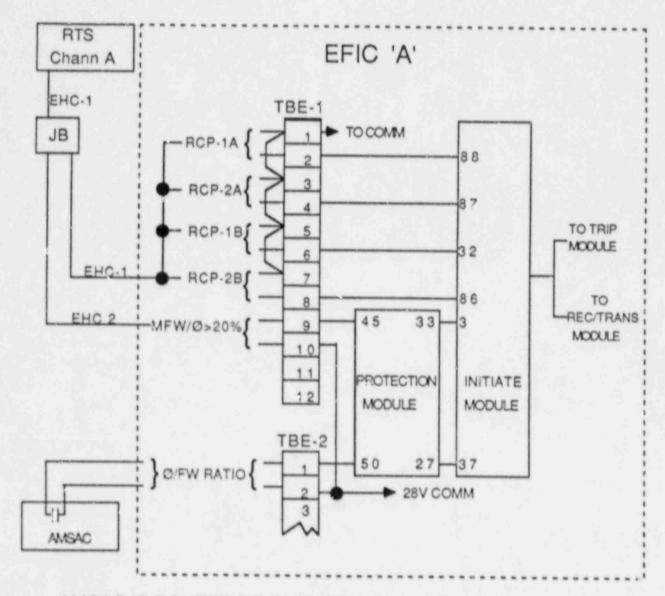


ENERGENCY FEEDWALER INITIATION AND CONTROL SYSTEM (EFIC)
CHANNEL D SIMPLIFIED DIAGRAM
FIGURE 7-26 SHT 4 OF 6
(REV 8)



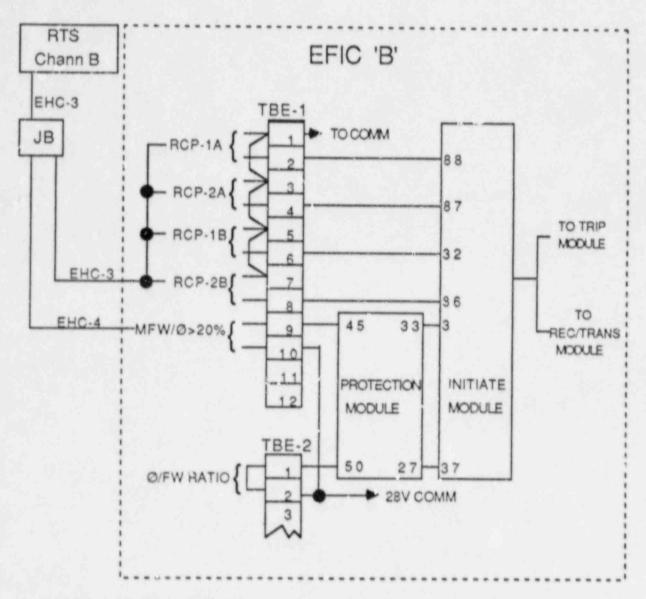
EMERGENCY FEEDWATER INITIATION AND CONTROL SYSTEM (EFIC) INITIATION LOGIC SIMPLIFIED DIAGRAM FIGURE 7-26 SH1 6 OF 6 (REV 8)





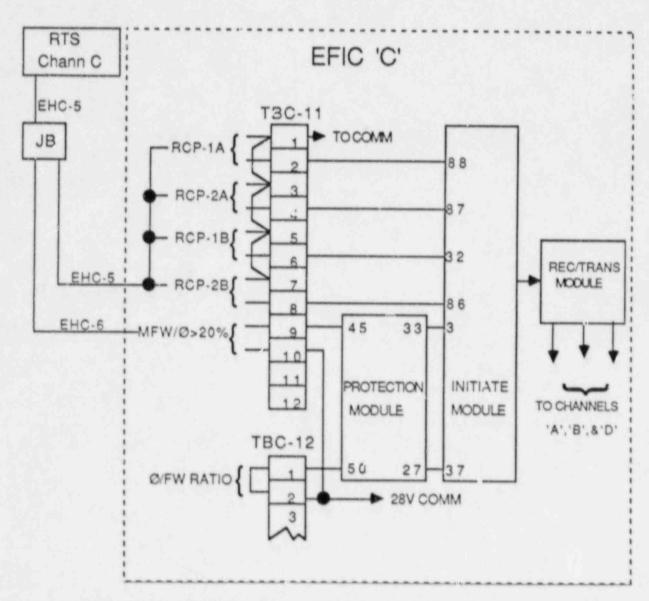
AMSAC IS ROUTED TO ONLY EFIC CHANNELS 'A' &'D'.

ACTUATION LOGIC CHANNEL 'A' = AC+AD+BC+BD

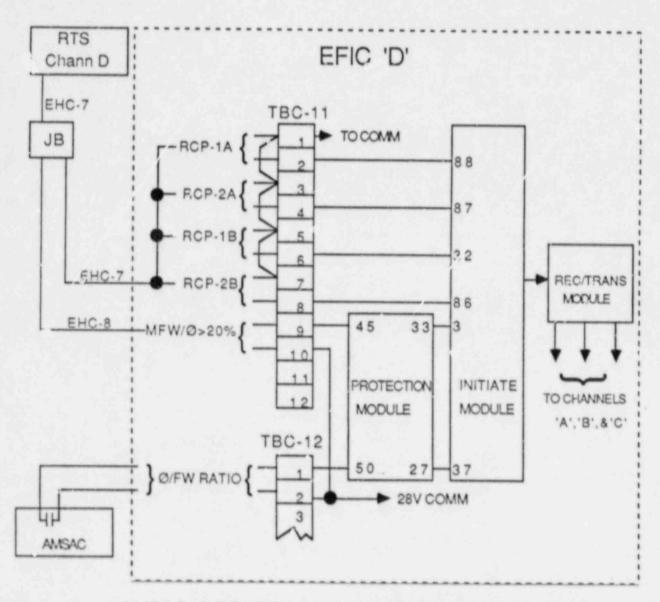


AMSAC IS ROUTED TO ONLY EFIC CHANNELS 'A' &'D'.

ACTUATION LOGIC CHANNEL 'B' = AB+AD+BC+↑D

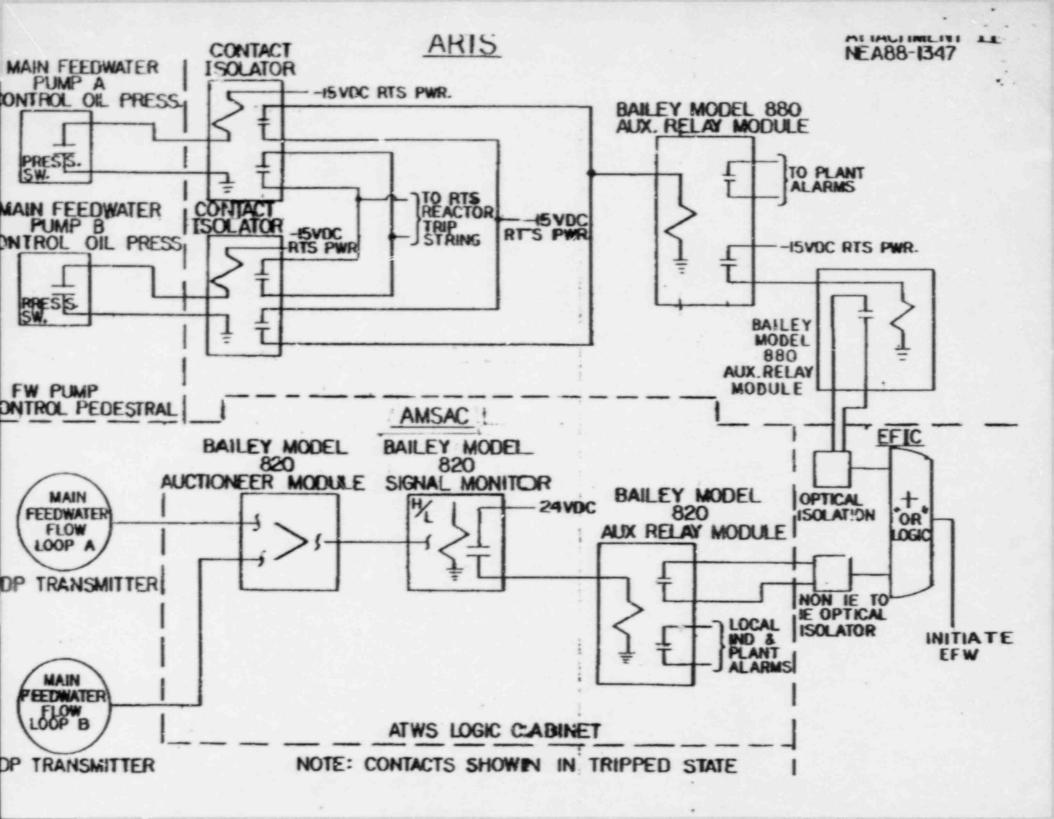


AMSAC IS ROUTED TO ONLY EFIC CHANNELS 'A' &'D'.



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AMSAC IS ROUTED TO ONLY EFIC CHANNELS 'A' &'D'.



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#### DISTRIBUTION FOR MEETING SUMMARY DATED:

Facility: Crystal River Unit 3

Docket File
NRC & Local PDRs
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NRC Participants
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L. Reyes, RII
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