U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO 3150-0104 EXPIRES B/31 MA LICENSEE EVENT REPORT (LER) DOCKET NUMBER (2) FACILITY NAME (1) Prairie Island Unit 1 0 15 10 10 10 1 2 1 8 1 2 1 OF 014 TITLE (4) Auto Start of 12 Component Cooling Water Pump OTHER FACILITIES INVOLVED (8) EARCH! DATE (7) EVENT DATE (8) LER NUMBER IS DOCKET NUMBERIS FACILITY NAMES SEQUENTIAL MONTH DAY MONTH DAY YEAR YEAR 0 15 10 10 10 1 40 119 81 7 2 0 0 01 8 0 15 10 10 10 1 THIS REPORT IS SUBMITTED PURSUANT TO THE NECESIFIREMENTS OF 10 CFR \$ (Check one or more of the following) (11) SPERATING 73.71(b) 20 A05(e) 50 73(a)(2)(iv) 73.71(e) 50 38(a)(1) 50.73(a)(2)(v) 30 408(a)(1)(i) OTHER (Specify in Abstract below and in Text, NAC Fo. 366A) 50.73(a)(2)(vii) 50 36(a)(2) 11 01 0 20.406(4)(1)(9) 50 73(a) (2) (viii) (A) 20 408(+)(1)(4) 50.73(4)(2)(1) 50.73(a)(2)(viii)(8) 50.73(a)(2)(k) 50.73(a)(2)(x) 20.408(a)(:/(v) 50 T Jan 12 1760 LICENSEE CONTACT FOR THIS LER (12) ELEPHONE NUMBER NAME Arne Hunstad, Staff Engineer 8 8 2 Prairie Island Nuclear Conerating Plant COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) EPORTABLE TO NPROS MANUFAC REI JATABLE MANUFAC CAUSE SYSTEM COMPONENT SYSTEM COMPONENT CAUSE YEAR DAY SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE (15) N NO YES III yes, complete EXPECTED SUBMISSION DATE! ABSTRACT (Limit to 1400 species, i.e., approximately fifteen single-space typewritten lines (16) On October 19, 1987, with both Units operating at full power, motor operated valve MV 32121 (No. 12 Component Cooling Water Heat Exchanger Outler Valve) was closed as part of prework testing for periodic maintenance. Due co a procedural inadequacy, when MV-32121 was closed, No. 12 Component Cooling (CC) Water Pump automatically started due to

low discharge header pressure. Prior to closing MV-32121, the No. 12 CC Pump discharge header was being supplied and pressurized by No. 11 CC Pump. When MV-32121 was closed the No. 12 CC pump discharge line was isolated from the No. 11 CC pump and the pressure in the line dropped to the No. 12 CC pump auto-start setpoint.

MV-32121 was immediately reopened and No. 12 CC pump was stopped. Work was stopped and the subject prework test procedure was modified to require the manual starting of No. 12 CC pump prior to the cycling of MV-32121. Subsequent testing of component cooling heat exchanger outlet valves was performed with both CC pumps for the associated unit in operation.

This event had no safety significance.

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NRC Form 364A (9-83) LICENSEE EVEN	T REPORT (LER) TEXT CONTINU	JATION	APPROVED	U.S. NUCLEAR REGULATORY COMMISSION APPROVED DMS NO. 3150-0104 EXPIRES: 8/31/85	
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DESCRIPTION OF THE EVENT

At 1323 on October 19, 1987, with both Units operating at full power, prework testing was begun for a work request to perform periodic maintenance on the No. 12 Component Cooling Water Heat Exchanger motor operated outlet valve MV-32121 (EIIS Identifier 20). The first step of the prework procedure called for obtaining as found motor load and current traces for MV-32121 as it was cycled closed and back open. When MV-32121 was closed No. 12 Component Cooling (CC) Water Pump (EIIS Identifier P) automatically started due to low discharge header pressure. Prior to closing MV-32121, the No. 12 Component Cooling Water Pump discharge header was being supplied and pressurized by No. 11 Component Cooling Water Pump. When MV-32121 was closed the No. 12 CC pump discharge line was isolated from the No. 11 CC pump and the pressure in the line dropped the the No. 12 CC pump auto-start setpoint of 65 psig. MV-32121 was immediately reopened and No. 12 CC pump was stopped. Work under the subject work request was stopped until necessary procedure changes could be implemented.

This report is being submitted more than 30-days following the date of the event. Northern States Power Company originally believe that this event was not reportable as a safeguards system actuation. Further clarification was provided by the Senior NRC Resident inspector and NRC management which indicated that a report was necessary.

Refer to the attached system flow diagram.

CAUSE OF THE EVENT

The cause of the event was a procedural inadequacy which allowed the No. 12 CC pump discharge header to be isolated from the No. 11 CC pump discharge header without first starting No. 12 CC pump.

ANALYSIS OF THE EVENT

During this event all equipment operated as designed. The automatic start of No. 12 CC pump ensured that all safety related components requiring component cooling water were provided with adequate pressure and flow to perform their required function and that at no time was the health and safety of the public jeopardized. Because this event was deemed to involve the automatic start of an engineered safety feature component it is reportable pursuant to 10 CFR Part 50, Section 50.73(a)(2)(iv).

U.S. HUCLEAR REGULATORY COMMISSION NRC Form 366A LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85 DOCKET NUMBER (2) FACILITY NAME (1) LER NUMBER (6) PAGE (3) SEQUENTIAL Prairie Island Unit 1 2 8 7 9 21 0 010 Q 3 OF 014 0 |5 |0 |0 |0 |

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CORRECTIVE ACTIONS

Valve MV-32121 was immediately reopened following the automatic start of No. 12 CC pump. No. 12 CC pump was then stopped when adequate pressure and flow from the No. 11 CC pump to the No. 12 CC pump discharge header was assured. The prework test procedure for the subject work request was modified to require the manual starting of No. 12 CC pump prior to the cycling of MV-32121. In addition the procedures for testing the other three component cooling heat exchanger outlet valves were modified to require the respective CC pump to be running prior to cycling the valve. Subsequent testing of component cooling heat exchanger outlet valves was performed with both CC pumps for the associated unit in operation.

The investigative report for this event will be routed to the individuals involved and the system engineers for the component cooling system as a reminder of the potential for the automatic starts of component cooling pumps during discharge header low pressure conditions.

NRC Form 366A (9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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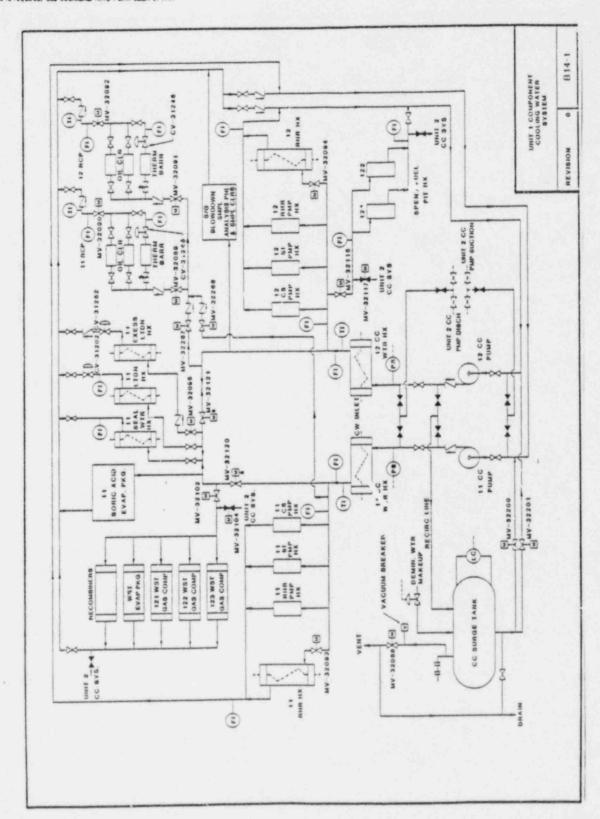
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Northern States Power Company

414 Nicollet Mali Minneapolis, Minnesota 55401 Telephone (612) 330-5500

February 4, 1988

10 CFR Part 50 Section 50.73

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

> PRAIRIE ISLAND NUCLEAR GENERATING PLANT Docket Nos. 50-282 License Nos. DPR-42 50-306 DPR-60

Auto Start of 12 Component Cooling Water Pump LER 1-87-020

The Licensee Event Report for this occurrence is attached.

This report is being submitted more than 30-days following the date of the event. Northern States Power Company originally believe that this event was not reportable as a safeguards system actuation. Further clarification was provided by the Senior NRC Resident inspector and NRC management which indicated that a report was necessary.

Please contact us if you require additional information related to this event.

David Musolf

Manager - Nuclear Support Services

c: Regional Administrator - III, NRC Sr Resident Inspector, NRC NRR Project Manager, NRC MPCA

Attn: Dr J W Ferman

Attachment

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