

June 20, 1988

Docket Nos. 50-259/260/296

APPLICANT: Tennessee Valley Authority

FACILITY: Browns Ferry Nuclear Plant, Units 1, 2 and 3

SUBJECT: SUMMARY OF MEETING HELD ON MAY 18, 1988 - SEISMIC QUALIFICATION
(CORRECTION)

In the above meeting summary dated June 9, 1988, the attached page was inadvertently omitted. The omitted page is the first page of Enclosure 2 and should follow the list of attendees (Enclosure 1) in the meeting summary.

Original Signed by

Janet L. Kelly, Project Engineer
TVA Projects Division
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Attachment:
As stated,

cc w/attachment:
See next page

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REVIEW COMMENTS ON BFN

INTERIM CRITERIA FOR SEISMIC DESIGN PROGRAM
(DISCUSSION TOPICS FOR MAY 18, 1988 MEETING)

I. Drywell Steel Platforms

References:

1. Letter from R. Gridley (TVA) to NRC, "BFN - Seismic Qualification of Drywell Steel," dated March 10, 1988.
2. Letter from R. Gridley (TVA) to NRC, "BFN - Seismic Qualification of Drywell Steel - (NRC TAC No. 00302)," dated April 28, 1988.
4. Enclosure 1 and Table 1 of Enclosure 1:
 1. 3rd sentence of Item 2 under Resolution Section - How were these loads generated and applied to the platform model?
 2. Items 3, 5 and 6, under Resolution Section - If modifications are needed as a result of evaluation, why are the interim operability criteria instead of long term criteria (or design criteria) to be used for the design of modifications.
 3. Licensing Issue - Provide basis for changing the factor "1.6" to "1.7" (References 1 and 2).
 4. Item 3 of "Justification" -
 - (i) NRC NUREG-0800, "Standard Review Plan," was issued after BFN was licensed. It is the staff's position that this document should be used as a whole package instead of piece by piece.
 - (ii) The staff treated the Mark I Torus Long Term Integrity Program as a case by case evaluation. Therefore, the criteria accepted under this program are not applicable for other evaluation, e.g., BFN Seismic Design Program.
5. Table 1 -
 - (i) The allowable stresses for steel and weld are given as a range because of the use of "up to." Specific allowable stresses should be explicitly defined for each of different load combinations to be considered in the re-start evaluation.
 - (ii) The "similar to" in the remark for the concrete anchor criteria is unspecified and should be clearly defined.
 - (iii) What are the allowables for steel members under compression and bending.
 - (iv) What are the allowable tension and shear stresses for bolts.

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Browns Ferry Nuclear Plant
Units 1, 2, and 3

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DISTRIBUTION FOR MEETING SUMMARY DATED: June 20, 1988

Facility: Browns Ferry Nuclear Plant, Units 1, 2 and 3*

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