

**AVERAGE DAILY UNIT POWER LEVEL**

DOCKET NO. 50-346

UNIT Davis-Besse Unit 1

DATE 3-88

COMPLETED BY J. Cipriani

TELEPHONE Ext. 7365

MONTH December 1987

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>700</u>	17	<u>690</u>
2	<u>699</u>	18	<u>689</u>
3	<u>679</u>	19	<u>685</u>
4	<u>663</u>	20	<u>683</u>
5	<u>693</u>	21	<u>683</u>
6	<u>701</u>	22	<u>689</u>
7	<u>203</u>	23	<u>694</u>
8	<u>-0-</u>	24	<u>696</u>
9	<u>462</u>	25	<u>694</u>
10	<u>695</u>	26	<u>695</u>
11	<u>691</u>	27	<u>695</u>
12	<u>692</u>	28	<u>695</u>
13	<u>693</u>	29	<u>695</u>
14	<u>694</u>	30	<u>693</u>
15	<u>691</u>	31	<u>694</u>
16	<u>692</u>		

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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**OPERATING DATA REPORT**

DOCKET NO. 50-346  
 DATE 1-13-88  
 COMPLETED BY J. Cipriani  
 TELEPHONE Ext. 7365

OPERATING STATUS

1. Unit Name: Davis Besse Unit 1
2. Reporting Period: December 1987
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 975
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 904
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): Continued at approximately 80 percent
10. Reasons For Restrictions, If Any: Due to the removal of two and the gagging of a third main steam safety valve on October 10, 1987.

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	8,760	82,656
12. Number Of Hours Reactor Was Critical	711.1	7,425.7	43,480.8
13. Reactor Reserve Shutdown Hours	32.9	425.3	5,050.1
14. Hours Generator On-Line	701.5	7,312.4	41,801
15. Unit Reserve Shutdown Hours	0	0	1,732.5
16. Gross Thermal Energy Generated (MWH)	1,542,784	16,535,534	9,962,199
17. Gross Electrical Energy Generated (MWE)	510,883	5,413,416	32,375,803
18. Net Electrical Energy Generated (MWH)	478,556	5,063,984	30,300,647
19. Unit Service Factor	94.3	83.5	50.6
20. Unit Availability Factor	94.3	83.5	52.7
21. Unit Capacity Factor (Using MDC Net)	74.8	67.2	42.6
22. Unit Capacity Factor (Using DER Net)	71.0	63.8	40.5
23. Unit Forced Outage Rate	5.7	6.8	33.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling - Start on March 11, 1988 - 26 weeks - End on September 11, 1988

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346  
 UNIT NAME Davis-Besse 1  
 DATE 1-13-88  
 COMPLETED BY J. Cipriani  
 TELEPHONE (419) 249-5000  
 ext. 7365

REPORT MONTH December 1987

* No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	* Cause & Corrective Action to Prevent Recurrence
12	87 07 12	F	42.5	A	3	87-015	LD	PSV	The Reactor tripped due to loss of Instrument Air. The cause of the loss of instrument air was the failure of Valve IA26 to close. (See operational summary for further details)

<sup>1</sup> F: Forced  
 S: Scheduled

<sup>2</sup> Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup> Method:  
 1-Manual  
 2-Manual Scram  
 3-Automatic Scram  
 4-Continuation from  
 Previous Month  
 5-Load Reduction  
 9-Other (Explain)

<sup>4</sup> Exhibit C - Instructions for Preparation of Data  
 Entry Sheets for Licensee Event Report (LER)  
 File (NUREG-0161)

<sup>5</sup> Exhibit I - Same Source  
 \*Report challenges to Power Operated Relief Valves  
 (PORVs) and Pressurizer Code Safety Valves (PCSVs)

## OPERATIONAL SUMMARY

December 1987

The reactor power was maintained at approximately 81% power until 2000 hours on December 3, 1987 when axial power shaping rods (APSR) were withdrawn as required by Technical Specifications APSR Insertion Limit. Reactor power was reduced to approximately 60% until 0300 hours on December 4, 1987 when power was increased to approximately 81%.

The reactor power was maintained at approximately 81% power until 0656 hours on December 7, 1987 when a reactor trip occurred. The reactor tripped due to the loss of Instrument Air (IA). The cause of the loss of IA was the failure of IA26 (Solenoid Operated Valve) to close. With IA26 open, Instrument Air supply pressure was diverted through the blowdown line.

The reactor criticality was established at 1550 hours on December 8, 1987.

The turbine generator was synchronized on line at 0125 hours on December 9, 1987.

Reactor power was slowly increased to approximately 80% power and maintained at this power level for the rest of the month. Reactor power was limited at 80% due to the removal of two mainsteam safety valves and gagging of a third main steam safety valve.



January 13, 1988  
KB88-00003

Docket No. 50-34'  
License No. NPF-3

Document Control Desk  
U. S. Nuclear Regulatory Commission  
7920 Norfolk Avenue  
Bethesda, MD 20555

Gentlemen:

Monthly Operating Report, December 1987  
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit 1 for the month of December 1987.

If you have any questions, please feel free to contact Bilal Sarsour at (419) 249-5000, Extension 7384.

Very truly yours,

A handwritten signature in cursive script that reads 'L F Storz / NRB'.

Louis F. Storz  
Plant Manager  
Davis-Besse Nuclear Power Station

LFS.GAG:ECC:BMS:plg

Enclosures

cc: Mr. A. Bert Davis, w/1  
Regional Administrator, Region III

Mr. Paul Byron, w/1  
NRC Resident Inspector

Nuclear Records Management, Stop 3220

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REFUELING INFORMATION

Date: December 1987

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling outage? March 1988
3. Scheduled date for restart following refueling: September 1988
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: Expect the Reload Report to require standard reload fuel design Technical Specifications changes (2. Safety Limits and Limiting Safety System Settings, 3/4.1 Reactivity Control Systems, 3/4.2 Power Distribution Limits and 3/4.4 Reactor Coolant System.)

5. Scheduled date(s) for submitting proposed licensing action and supporting information: February 1988
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
  1. The highly absorbing silver-indium-cadmium axial power shaping rods will be replaced with reduced absorbing inconel rods.
  2. The discrete neutron sources will be removed from the core and not replaced.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.  
(a) 177                      (b) 204 - Spent Fuel Assemblies
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.  
Present: 735                      Increased size by: 0 (zero)
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1995 - assuming ability to unload the entire core into the spent fuel pool is maintained

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