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the southern electric system

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June 14, 1988

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

PLANT VOGTLE - UNIT 1
NRC DOCKET 50-424
OPERATING LICENSE NPF-68
REQUEST TO REVISE
TECHNICAL SPECIFICATION 6.3 - TRAINING

Gentlemen:

In accordance with 10 CFR 50.90 as required by 10 CFR 50.59(c)(1), Georgia Power Company (GPC) hereby proposes to amend the Vogtle Electric Generating Plant Unit 1 Technical Specifications, Appendix A to Operating License NPF-68.

The proposed amendment incorporates recent changes to 10 CFR 55 into the licensing program.

Enclosure 1 provides a detailed description of the proposed changes and the bases for the changes.

Enclosure 2 details the basis for our determination that the proposed changes do not involve significant hazards considerations.

Enclosure 3 provides instructions for incorporating the proposed changes into the Technical Specifications. The proposed correct page follows Enclosure 3.

In accordance with 10 CFR 170.12, a check for payment of the \$150.00 license amendment filing fee is enclosed.

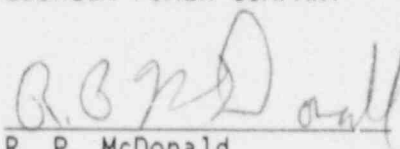
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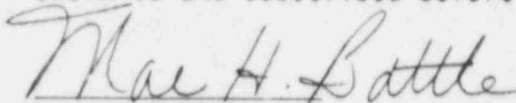
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Mr. R. P. McDonald states that he is Executive Vice President of Georgia Power Company and is authorized to execute this oath on behalf of Georgia Power Company, and that to the best of his knowledge and belief, the facts set forth in this letter and enclosures are true.

GEORGIA POWER COMPANY

By: 
R. P. McDonald

Sworn to and subscribed before me this 14th day of June, 1988.



Notary Public

Notary Public, Fulton County, Ga.
My Commission Expires Nov. 2, 1991

PAH/lm

Enclosures:

1. Basis for Proposed Changes
2. 10 CFR 50.92 Evaluation
3. Instructions for Incorporation
4. Check No. 044779 for \$150.00 filing fee

c: Georgia Power Company
Mr. P. D. Rice
Mr. G. Bockhold, Jr.
GO-NORMS

U. S. Nuclear Regulatory Commission
Dr. J. N. Grace, Regional Administrator
Mr. J. B. Hopkins, Licensing Project Manager, NRR (2 copies)
Mr. J. F. Rogge, Senior Resident Inspector-Operations, Vogtle

ENCLOSURE 1

PLANT VOGTLE - UNIT 1
NRC DOCKET 50-424
OPERATING LICENSE NPF-68
REQUEST TO REVISE TECHNICAL SPECIFICATION 6.3
BASIS FOR PROPOSED CHANGES

PROPOSED CHANGE

Revise Paragraph 6.3.1 to read as follows:

6.3 Training

6.3.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the Plant Training and Emergency Preparedness Manager. Personnel will meet the minimum education and experience recommendations of ANSI N18.1-1971 and, for licensed staff, 10 CFR 55.59 and the supplemental requirement specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, before they are considered qualified to perform all duties independently. Prior to meeting the recommendation of ANSI N18.1-1971, personnel may be trained to perform specific tasks and will be qualified to perform those tasks independently. Personnel who complete an accredited program which has been endorsed by the NRC shall meet the requirements of the accredited program in lieu of the above.

BASIS

In Generic Letter 87-07, it was stated that the 55.59(c) content requirements (formerly Appendix A to 10 CFR 55) can be met with a performance-based program for a facility as approved by the NRC. In its Final Policy Statement on Training and Qualification of Nuclear Power Plant Personnel, the Commission endorsed industry-accredited programs as performance based.

In the Federal Register Volume 52 No. 57 dated Wednesday, March 25, 1987, it was stated that:

"The requalification program must meet the requirements of paragraphs (c)(1) through (7) of this section. In lieu of paragraphs (c)(2), (3), and (4) of this section, the Commission may approve a program developed by using a systems approach to training."

ENCLOSURE 1 (Continued)

REQUEST TO REVISE TECHNICAL SPECIFICATION 6.3
BASIS FOR PROPOSED CHANGES

The proposed change to the training sections reflects the sentiments of the Commission by allowing the use of a performance based industry-accredited program for training of the licensed staff.

ENCLOSURE 2

PLANT VOGTLE - UNIT 1
NRC DOCKET 50-424
OPERATING LICENSE NPF-68
REQUEST TO REVISE TECHNICAL SPECIFICATION 6.3
10 CFR 50.92 EVALUATION

In accordance with 10 CFR 50.92, Georgia Power Company has evaluated the attached proposed amendment to the VEGP Unit 1 Technical Specifications and has determined that operation of the facility in accordance with the proposed amendment would not involve significant hazards considerations.

BACKGROUND

The purpose of this technical specification change is to mirror the changes in requalification program requirements that recently appeared in the Code of Federal Regulations, Vol. 52 No. 57, Wednesday, March 25, 1987. The code now allows the Commission to approve a program developed by using a systems approach to training.

1. The proposed change does not significantly increase the probability or consequences of an accident previously evaluated. This change allows recognition of a training program that has received an NRC endorsement or approval. Since the plant operating staff will be "approved", the probability of any previously evaluated accident is not affected. The consequences of previously analyzed accidents are not significantly increased.
2. The proposed change does not create the possibility of a new or different kind of accident than any accident previously evaluated. The plant operating staff is still subject to an NRC approved program. A new or different failure mode should not result from the training given to the operating staff.
3. The proposed change does not involve a significant reduction in the margin of safety. Plant operations personnel, both before and after the change, operated the plant subject to NRC review and approval of their training. In this case, the NRC is approving or endorsing a performance based training program. Safety margins are therefore not reduced since the operation training program is still subject to NRC review and approval.

ENCLOSURE 2 (Continued)

REQUEST TO REVISE TECHNICAL SPECIFICATION 6.3
10 CFR 50.92 EVALUATION

CONCLUSION

Based on the preceding analysis, we have determined that the proposed change to the Technical Specifications will not significantly increase the probability or consequences of an accident previously evaluated, create the possibility of a new or different kind of accident from any accident previously evaluated, or involve a significant reduction in a margin of safety. It has been concluded that the proposed change meets the requirements of 10 CFR 50.92(c) and does not involve a significant hazards consideration.