

APPENDIX

U. S. NUCLEAR REGULATORY COMMISSION
REGION IV

NRC Inspection Report: 50-482/88-16

Operating License: NPF-42

Docket: 50-482

Licensee: Wolf Creek Nuclear Operating Corporation (WCNOC)
P. O. Box 411
Burlington, Kansas 66839

Facility Name: Wolf Creek Generating Station (WCGS)

Inspection At: Wolf Creek Site, Coffey County, Burlington, Kansas

Inspection Conducted: April 1 through May 14, 1988

Inspectors: *Wright J. Chamberlain for* *6-6-88*
B. L. Bartlett, Senior Resident Inspector,
Project Section A, Division of Reactor
Projects Date

Wright J. Chamberlain for *6-6-88*
M. E. Skow, Resident Reactor Inspector,
Project Section A, Division of Reactor
Projects Date

Approved: *Wright J. Chamberlain* *6-6-88*
D. D. Chamberlain, Chief, Project Section A,
Division of Reactor Projects Date

Inspection SummaryInspection Conducted April 1 through May 14, 1988 (Report 50-482/8816)

Areas Inspected: Routine, unannounced inspection including follow up of previously identified inspection findings, monthly surveillance observation, monthly maintenance observation, operational safety verification, onsite event followup, radiological protection, and physical security verification.

Results: Within the areas inspected, no violations or deviations were identified.

DETAILS1. Persons ContactedWCNOC

- R. M. Grant, Vice President, Quality Assurance (QA)
- *G. D. Foyer, Plant Manager
- *O. L. Maynard, Manager, Licensing
- *C. M. Estes, Superintendent of Operations
- M. D. Rich, Superintendent of Maintenance
- *M. G. Williams, Superintendent of Plant Support
- *A. A. Freitag, Manager, Nuclear Plant Engineering (NPE), WCGS
- *K. Peterson, Licensing
- *G. Pendergrass, Licensing
- *W. M. Lindsay, Supervisor, Quality Systems
- *C. J. Hoch, QA Technologist
- J. A. Zell, Training Manager
- *E. Lehmann, NSE Engineer
- *J. Pippin, Manager, Nuclear Plant Engineering
- *R. W. Holloway, Manager, Maintenance and Modifications
- *C. E. Parry, Manager, QA

The NRC inspectors also contacted other members of the licensee's staff during the inspection period to discuss identified issues.

*Denotes those personnel in attendance at the exit meeting held on May 25, 1988.

2. Follow Up on Previously Identified Inspection Findings (92701)

- a. (Closed) Unresolved Item (482/8701-02): Control Room Operating Curves - This unresolved item concerned licensee actions to verify that all information going into WCRX-04, Revision 2, "Control Room Operating Curves and Tables Cycle 2," and revisions to WCRX-04 were made through plant safety review committee reviewed and approved procedures. The NRC inspector has verified that WCRX-04 did receive the appropriate review and approval prior to implementation. This unresolved item is closed.
- b. (Closed) Violation (482/8622-02): Failure to Control Documents Marked as Safeguards Information - This violation concerned the failure to properly control documents that were marked as safeguards information (SI). The licensee destroyed all of the improperly controlled copies, verified the papers were not SI, and counseled the personnel involved. This violation is closed.

3. Monthly Surveillance Observation (61726)

The NRC inspectors observed selected portions of the performance of surveillance testing and/or reviewed completed surveillance test procedures to verify that surveillance activities were performed in accordance with Technical Specification (TS) requirements and administrative procedures. The NRC inspectors considered the following elements while inspecting surveillance activities:

- o Testing was being accomplished by qualified personnel in accordance with an approved procedure.
- o The surveillance procedure conformed to TS requirements.
- o Required test instrumentation was calibrated.
- o Technical Specification limiting conditions for operation (LCO) were satisfied.
- o Test data was accurate and complete. The NRC inspectors performed independent calculations of selected test data in order to verify accuracy.
- o The performance of the surveillance procedure conformed to applicable administrative procedures.
- o The surveillance was performed within the required frequency and the test results met the required limits.

Surveillance witnessed and/or reviewed by the NRC inspectors are listed below:

- o STS KJ-005A, Revision 11, "Manual/Auto Start, Synchronization, and Loading of Emergency Diesel Generator NEC1," performed on May 4, 1988.
- o STS RE-012, Revision 1, "QPTR Determination," performed on May 4, 1988.
- o STS SE-002, Revision 5, "Manual Calculation of Reactor Thermal Power," performed on May 4, 1988.
- o STS PE-054, Revision 1, "Flow Rate Verification of Fans With Monitored Exhausts," performed on May 4, 1988.
- o STS SF-001, Revision 4, "Control and Shutdown Rod Operability Verification," performed on May 5, 1988.

No violations or deviations were identified.

4. Monthly Maintenance Observation (62703)

The NRC inspector observed maintenance activities performed on safety-related systems and components to verify that these activities were conducted in accordance with approved procedures, TS, and applicable industry codes and standards. The following elements were considered by the NRC inspector during the observation and/or review of the maintenance activities:

- o LCOs were met and, where applicable, redundant components were operable.
- o Activities complied with adequate administrative controls.
- o Where required, adequate, approved, and up-to-date procedures were used.
- o Craftsmen were qualified to accomplish the designated task and technical expertise (i.e., engineering, health physics, operations) was made available when appropriate.
- o Replacement parts and materials being used were properly certified.
- o Required radiological controls were implemented.
- o Fire prevention controls were implemented where appropriate.
- o Required alignments and surveillances to verify post-maintenance operability were performed.
- o Quality control hold points and/or checklists were used when appropriate and quality control personnel observed designated work activities.

Selected portions of the maintenance activities accomplished on the work requests (WR) listed below were observed and related documentation was reviewed by the NRC inspector:

<u>No.</u>	<u>Activity</u>
WR 01606-88	Isolation Damper GKD080 - bad damper wedge
WR 50755-88	Engineered Safety Features Transformers XNB01/XNB02 monthly inspection
WR 01421-88	Feedwater Isolation Valve AE FV-039 hydraulic fluid leak
WR 01844-88	Containment Cocier "A" SGN01A-no CMTR available for brazing material

WR 01874-88

Containment Cooler "A" SGN01A-repairs performed without qualified procedures

No violations or deviations were identified in this area of inspection.

5. Operational Safety Verification (71707)

The NRC inspectors verified that the facility is being operated safely and in conformance with regulatory requirements by direct observation of licensee facilities, tours of the facility, interviews and discussions with licensee personnel, independent verification of safety system status and limiting conditions for operations, and reviewing facility records. The NRC inspectors, by observation of randomly selected activities and interviews of personnel, verified that physical security, radiation protection, and fire protection activities were controlled.

By observing accessible components for correct valve position and electrical breaker position, and by observing control room indications, the NRC inspectors confirmed the operability of selected portions of safety-related systems. The NRC inspector also visually inspected safety-related components for leakage, physical damage, and other impairments that could prevent them from performing their designed functions.

Selected NRC inspector observations in this area are discussed below:

- a. NRC Temporary Instruction 2515/93 was performed as a portion of this area of the inspection. The purpose of this instruction was to verify that the licensee had complied with 10 CFR Part 50, Appendix B, Quality Assurance requirements regarding diesel generator fuel oil as identified in NRC Multi-Plant Action Item A-15. The NRC inspector reviewed TS 4.8.1.1.2b, c, d, and e for surveillance requirements for emergency diesel generator fuel oil. The following procedures were reviewed and found to meet the technical specification surveillance requirements:
 - o STS JE-003A, Revision 3, "Diesel Generator NE01 Day Tank Water Removal"
 - o STS JE-003B, Revision 3, "Diesel Generator NE02 Day Tank Water Removal"
 - o STS JE-004A, Revision 4, "Emergency Fuel Oil Storage Tank Water Check/Removal"
 - o STS JE-004B, Revision 3, "Emergency Fuel Oil Storage Tank Water Check/Removal"
 - o STS CH-015, Revision 1, "Emergency Diesel New Fuel"

- o STS CH-008A, Revision 5, "Emergency Diesel Fuel Storage Tank TJE01A"
- o STS CH-008B, Revision 5, "Emergency Diesel Fuel Storage Tank TJE01B"

The licensee has included diesel generator fuel oil in their quality assurance program. This TI is closed.

- b. During a routine tour of the auxiliary building on April 28, 1988, the NRC inspector observed the following outdated information tags still attached to some pieces of equipment:
 - o Non-Destructive Examination Request Tag 100780, dated March 8, 1984, was found hanging from reactor coolant pump "C" seal water supply line.
 - o Equipment Removal Tag 27361, dated October 29, 1984, was found hanging from Valve BG HV-8351C.
 - o A warehouse preventive maintenance tag dated October 1984, was also found hanging from Valve BG HV-8351C.

The licensee was given this information for removal of tags when they are no longer applicable.

- c. On April 5, 1988, the licensee informed the NRC resident inspector of Defect/Deficiency Report (DDR) 88-015. This DDR documented the discovery on June 29, 1987, that four terminal boxes did not have drain holes. These drain holes were required for equipment qualification purposes. These terminal boxes are inside containment. The terminal boxes involved were EJZS 8701BA, BBHV8001A, BBHV8001B, BBHV8002A, and BBHV8002B. Work Request 2449-87 dated July 8, 1987, documented a nuclear plant engineering (NPE) evaluation of the lack of drain holes. The NPE evaluation found that it could be postulated that a flooding event could have rendered the four reactor head vent valves inoperable. Updated Safety Analysis Report Table 3.11(B)-3 states that the valves are required to be functional for post loss-of-coolant accidents and post main steamline break accidents. On June 29, 1987, the licensee drilled the required holes and, using Plant Modification Request 02352, revised the applicable drawings. This matter will be evaluated by NRC Region IV along with the first round inspection equipment qualification issues at Wolf Creek and will be tracked as an open item pending the evaluation. (482/8816-01)

6. Onsite Event Followup (92700)

The NRC inspectors performed onsite followup of a nonemergency event that occurred during this report period. The NRC inspectors reviewed control

room logs and discussed the event with cognizant personnel. The NRC inspectors verified the licensee had responded to the event in accordance with procedures and had notified the NRC and other agencies as required in a timely fashion. The event that occurred during this report period is listed in the table below. The NRC inspectors will review the LER for this event and will report any findings in a subsequent NRC inspection report.

<u>Date</u>	<u>Event</u>	<u>Plant Status</u>	<u>Cause</u>
4/2/88	CRVIS*	Mode 1	Spike on GK AIT-2

*CRVIS - Control room ventilation isolation

In addition to the above event, a problem with the loose part monitoring system was reviewed on April 14, 1988, the licensee informed the NRC inspectors of their failure to properly perform TS surveillance requirement 4.3.3.9.a on the loose part monitoring system (LPMS). The licensee formally notified the NRC of this failure in Licensee Event Report 88-004 dated May 16, 1988, in accordance with 10 CFR 50.73(a)(2)(i)(B). TSs require that a daily channel check be performed on the LPMS and the licensee identified that the daily channel check that was being performed was inadequate. An additional followup inspection was performed by NRC region based inspectors. Their conclusions are documented in NRC Inspection Report 50-482/88-17.

7. Radiological Protection (71709)

By performing the following activities, the NRC inspectors verified that radiologically related activities were controlled in accordance with the licensee's procedures and regulatory requirements:

- o Reviewed documents such as active radiation work permits and the health physics shift turnover log.
- o Observed personnel activities in the radiologically controlled area (RCA) such as:
 - . Use of the required dosimetry equipment,
 - . "Frisking out" of the RCA, and
 - . Wearing of appropriate anti-contamination clothing where required.
- o Inspected postings of radiation and contaminated areas.
- o Discussed activities with radiation workers and health physics supervisors.

No violations or deviations were identified.

8. Physical Security Verification (71881)

The NRC inspectors verified that the facility physical security plan (PSP) is being complied with by direct observation of licensee facilities and security personnel.

The NRC inspectors, by observation of randomly selected activities, verified that search equipment is operable, that the protected area barriers and vital area barriers are well maintained, that access control procedures are followed, and that appropriate compensatory measures are followed when equipment is inoperable.

No violations or deviations were identified.

9. Exit Meeting

The NRC inspectors met with licensee personnel to discuss the scope and findings of this inspection on May 25, 1988.