

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 86-354

UNIT Hope Creek

DATE 6/15/88

COMPLETED BY H. Jensen

TELEPHONE (609) 339-5261

MONTH May 1988

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0</u>
2	<u>0</u>
3	<u>607</u>
4	<u>1044</u>
5	<u>361</u>
6	<u>396</u>
7	<u>1017</u>
8	<u>1017</u>
9	<u>1008</u>
10	<u>1031</u>
11	<u>1030</u>
12	<u>998</u>
13	<u>1013</u>
14	<u>1009</u>
15	<u>1013</u>
16	<u>1018</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>1017</u>
18	<u>1012</u>
19	<u>1005</u>
20	<u>1024</u>
21	<u>1012</u>
22	<u>1013</u>
23	<u>994</u>
24	<u>1007</u>
25	<u>1021</u>
26	<u>1020</u>
27	<u>1013</u>
28	<u>1025</u>
29	<u>989</u>
30	<u>1010</u>
31	<u>1000</u>

OPERATING DATA REPORT

DOCKET NO. 86-354
 UNIT Hope Creek
 DATE 6/15/88
 COMPLETED BY H. Jenser *HJ*
 TELEPHONE (609) 339-5261

OPERATING STATUS

1. REPORTING PERIOD May 1988 GROSS HOURS IN REPORTING PERIOD 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt) 3293
 MAX. DEPEND. CAPACITY (MWe-Net) 1067 (1)
 DESIGN ELECTRICAL RATING (MWe-Net) 1067 (1)
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net) None
4. REASONS FOR RESTRICTION (IF ANY)
5. NO. OF HOURS REACTOR WAS CRITICAL

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NO. OF HOURS REACTOR WAS CRITICAL	<u>688.7</u>	<u>2216.1</u>	<u>10,074.2</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0</u>	<u>0</u>	<u>0</u>
7. HOURS GENERATOR ON LINE	<u>666.8</u>	<u>2060.0</u>	<u>9805.1</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0</u>	<u>0</u>	<u>0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>2,165,136</u>	<u>6,668,991</u>	<u>30,477,559</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	<u>714,205</u>	<u>2,217,173</u>	<u>10,128,871</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH)	<u>683,964</u>	<u>2,107,835</u>	<u>9,672,873</u>
12. REACTOR SERVICE FACTOR	<u>92.6</u>	<u>60.8</u>	<u>79.4</u>
13. REACTOR AVAILABILITY FACTOR	<u>92.6</u>	<u>60.8</u>	<u>79.4</u>
14. UNIT SERVICE FACTOR	<u>89.6</u>	<u>56.5</u>	<u>77.2</u>
15. UNIT AVAILABILITY FACTOR	<u>89.6</u>	<u>56.5</u>	<u>77.2</u>
16. UNIT CAPACITY FACTOR (Using Design MDC)	<u>86.2</u>	<u>54.2</u>	<u>71.4</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)	<u>86.2</u>	<u>54.2</u>	<u>71.4</u>
18. UNIT FORCED OUTAGE RATE	<u>10.4</u>	<u>4.3</u>	<u>8.0</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION):	<u>N/A</u>		
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:	<u>N/A</u>		

(1) August 1987 data is under management review.

OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 86-354

UNIT Hope Creek

DATE 6/15/88

COMPLETED BY H. Jensen

REPORT MONTH May, 1988

TELEPHONE (609) 339-5261

NO.	DATE	TYPE F FORCED S SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/ COMMENTS
5	5/1	F	50.4	A	4	CONTINUATION OF 4/30/88 FORCED OUTAGE
6	5/05	F	26.8	A	3	AUTOMATIC REACTOR SCRAM ON LOW REACTOR WATER LEVEL DUE TO "B" REACTOR FEED PUMP TURBINE TRIP. LER 88-013

SUMMARY

REFUELING INFORMATION

COMPLETED BY: Chris Brennan

DOCKET NO.: 50-354
UNIT NAME: Hope Creek Unit 1
DATE: 6/15/88
TELEPHONE: 3193
EXTENSION: N/A

Month May 1988

1. Refueling information has changed from last month:
YES _____ NO X
2. Scheduled date for next refueling: 11-04-89
3. Scheduled date for restart following refueling:
12-18-89
4. A) Will Technical Specification changes or other license amendments be required?
YES X NO _____
B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
YES _____ NO X
If no, when is it scheduled? 6-18-89
5. Scheduled date(s) for submitting proposed licensing action:
7-18-89
6. Important licensing considerations associated with refueling:
Information not presently available

7. Number of Fuel Assemblies:
A) Incore 764
B) In Spent Fuel Storage 232
8. Present licensed spent fuel storage capacity: 1108
Future spent fuel storage capacity: 4006
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 12-18-89

HOPE CREEK GENERATING STATION
MONTHLY OPERATING SUMMARY
MAY 1988

The unit remained in shutdown at the beginning of May due to the manual reactor scram which occurred on April 30. The generator was synchronized to the grid at 2:22 am on May 3. There was an automatic reactor scram at 9:17 am on May 5 on low reactor water level due to the "B" Reactor Feed Pump Turbine trip. The unit was returned to service on May 6 and was synchronized to the grid at 12:08 pm on that date. As of May 31, the unit has been online for 25 consecutive days.

R-011
RAR:tlb

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

MAY 1988

The following Design Change Packages (DCPs) have been evaluated to determine:

- 1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
- 2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
- 3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the DCPs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These DCPs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DCP

Description of Design Change Package

4-HMO-86-0514

This DCP removed the splash guards and their holdown bolts from the Reactor Recirculation Pumps. The splash guards were removed because of interference that was encountered involving vibration monitoring probes, thermocouples and the splash guards. It was determined that the splash guards did not fulfill a necessary function and could, therefore, be removed.

4EC-1059

This DCP permanently incorporated a Temporary Modification to the Reactor Feedwater Level Control system. This modification added a function generator in the Start-Up Level Control portion of the system. This DCP verified the proper installation of the function generator.

4EC-1082/11

This DCP labeled indicators and recorders in the Control Room. The labeling will assist the operators in determining which parameters are part of Accident-Monitoring Instrumentation or System Operation and Effluent-Monitoring Instrumentation.

4HM-0141

This DCP permanently installed 3 Temporary Modifications in the Filtration, Recirculation, and Ventilation System. The modification installed new flow sensors, temperature compensation resistors, and factory installed circuit modifications to provide increased stability.

4HM-0146

This DCP installed flanges and spool pieces on the steam and water supply lines to the Desuperheater in the Solid Radwaste system. This DCP allows maintenance to be performed on the Desuperheater without cutting and re-welding piping, which was required by the previous design.

4HM-0149

This DCP changed the alarm setpoints for the Generator Temperature High Annunciator to implement vendor recommendations and applicable operating experience. It also corrected the Recorder Analog Calibration Span.

4HM-0189

This was a document change only DCP to update drawings to reflect the deletion of the lantern rings in the packing gland of a control valve in the Safety Auxiliaries Cooling System. The lantern ring packing design represents an old technology and has been replaced with self-lubricating packing as the resolution for a Deficiency Report.

DCP

Description of Design Change Package

4HM-0239

This DCP added a time delay to the Instrument Air Dryer Circuitry to allow for momentary high flow conditions during Dryer Tower Cycle Swaps. It will also prevent a local High Flow Alarm from occurring during the swap.

4HM-0285

This DCP changed the Solid Radwaste Extruder Evaporator torque setpoints to allow the Extruder Evaporator to operate without spurious trips of the metering pumps. Changing the setpoints also allowed the high-high setpoint to be set. The previous high-high setpoint was at the top of the instrument range and did not allow for consistent calibration.

4HM-0303

This DCP was a component change out DCP which replaced the existing NA-200 type Local Power Range Monitors with a GE authorized replacement, the NA-300 detectors. The new model meets or exceeds the original design in fit, form and function.

4HM-0342

This DCP increased the reliability of the Control Room Chilled Water System by replacing the High-Side Ball Floats with High-Side Ball Floats with Reinforcement Pads. The new Ball Float is identical to the original in fit, form, and function; and meets or exceeds the existing component specifications.

The following Temporary Modification Requests (TMRs) have been evaluated to determine:

- 1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
- 2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
- 3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the TMRs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These TMRs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Safety EvaluationDescription of Temporary Modification Request (TMR)

- 88-0078 This TMR added a jumper across contacts for a defective High Oil Temperature Sensor until a replacement sensor is available. The sensor is located in the "B" Turbine Building Chiller, adding the jumper will allow the chiller to operate. An operator will monitor the associated temperature indicator while the TMR is in place and the chiller will be used as a back-up only during that time period.
- 88-0079 This TMR allowed for the installation of a temporary spool piece within the Cation Vessel of the Condensate Demineralizer system. The original spool piece required replacement; during the replacement it was discovered that the horizontal header was bent, necessitating a longer spool.
- 88-0080 This TMR installed a blank flange between 2 existing flanges in a line to a containment penetration. The additional flange reduced the leakage rate of the penetration and restored its integrity.
- 88-0081 This TMR installed an isolation valve in the supply line and a cap on the discharge line of the Emergency Bearing Oil Pump Low Operating Oil Pressure Test Solenoid Valve. The solenoid was leaking by, causing a low operating oil pressure signal which caused the pump to run continuously. The installation of the valve will allow repair of the solenoid when parts are available and the cap will eliminate the low pressure signal that starts the pump.
- 88-0082 This TMR installed a jumper around the Automatic Reset Switch for a Control Room Supply Vent Unit Temperature Switch. The temperature switch is designed to turn off the Control Room Supply Ventilation Heaters at 200°F and automatically resets at a lower value to prevent overheating of the heater coils while maintaining the Control Room within its designed environmental boundaries. The switch limits the temperature of the heater element only and is backed up by a manually reset trip at 250°F, which will function normally.

The following Deficiency Requests (DRs) have been evaluated to determine:

- 1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
- 2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
- 3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the DRs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These DRs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Safety Evaluation

Description of Deficiency Report (DR)

88-0072

This Safety Evaluation approves the use-as-is disposition of 2 DR's dealing with check valve welds. The preheat temperatures of these welds were not recorded on the Code Job Package forms. However, due to the location and function of these 2 valves, the failure to record the preheat temperatures did not adversely impact the ability of the valves to operate as designed.

88-0073

The torque wrenches used to torque pivot mount bolts on 1/4" and 1/2" mechanical snubbers have an operating range of 30-200 inch-pounds. However, the torque wrenches were adjusted to 22 inch-pounds because the vendor manual specified a torque range of 20-28 inch-pounds for these bolts. A sample of the bolts was re-checked and verified to be acceptable.

88-0075

A pivot mount attachment bolted to a snubber was incorrectly torqued to 22 inch-pounds. The vendor manual specifies a torque valve of 45 inch-pounds. The design torque valve was selected assuming a load larger than exists in the current application; therefore, a torque valve of 22 inch-pounds is acceptable in this application.