NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	1 OF 0 2

TITLE (4)

Engineered Safety Features Actuation Signal - Feedwater Isolation Signal

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 4	2 0	8 6	8 6	0 2 4	0 0	0 5	2 0	8 6		0 5 0 0 0

OPERATING MODE (9)	3
POWER LEVEL (10)	1 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: *(Check one or more of the following)* (11)

20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER *(Specify in Abstract below and in Text, NRC Form 366A)*
20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services	AREA CODE 3 1 6 — 3 6 4 - 8 8 3 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES *(If yes, complete EXPECTED SUBMISSION DATE)*	X NO				

ABSTRACT *(Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines)* (16)

On April 20, 1986, at 1208 CST, a Feedwater Isolation Signal (FWIS) and Main Turbine trip signal were initiated by a high-high water level condition in Steam Generator "D". At the time of this event, the unit was in Mode 3, Hot Standby, and steam dump to the condenser was being established. The Main Steamline Isolation Valve (MSIV) bypass valves were open and the MSIV associated with Steam Generator 'D' was partially open for testing when the level in Steam Generator 'D' momentarily increased to approximately 80 percent. This high-high level condition satisfied the minimum Engineered Safety Features actuation logic, resulting in the initiation of the FWIS and Main Turbine trip signal. Because the equipment affected by these signals was already in the required status, no equipment was actuated when these signals were generated.

This event was the result of licensed personnel overlooking the effect of increasing the steam demand while one MSIV was partially open. To prevent recurrence of this event, this report will be incorporated into required reading for operating personnel in order to highlight the sensitivity of Steam Generator levels in response to changes in steam demand.

There was no damage to plant equipment or release of radioactivity as a result of this event, and at no time did conditions develop that may have posed a threat to the health or safety of the public. There have been no previous similar occurrences.

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1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	8 6	— 0 2 4	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On April 20, 1986, at 1208 CST, a Feedwater Isolation Signal (FWIS) and Main Turbine trip signal were initiated by a high-high water level condition in Steam Generator (S/G) "D" [AB-SG].

At the time of the event, the unit was in Mode 3, Hot Standby, and the establishment of steam dump to the Main Condenser [SG-COND] was in progress. The four Main Steamline Isolation Valve (MSIV) bypass valves [SB-V] were open. The Main Steamline Isolation valve AB HV-11 [SB-ISV] associated with Steam Generator "D" had been fast stroke tested and left partially opened during testing. The steam dump demand had been adjusted to accomodate Reactor Coolant System [AB] heatup. The level in Steam Generator "D" momentarily increased to approximately 80 percent due to the effects of "swell". This high-high level condition satisfied the Engineered Safety Features Actuation logic, resulting in the initiation of the FWIS and Main Turbine trip signal.

Because the equipment affected by these signals was already in the required status, no equipment was actuated when these signals were generated. The Control Room operators almost immediately reduced Auxiliary Feedwater flow and Steam Generator "D" level returned to normal shortly thereafter.

The high-high level condition in Steam Generator "D" which initiated this event was the result of a cognitive personnel error by licensed personnel who overlooked the combined effects of several factors, including the initial level in Steam Generator "D", the establishment of steam dump to the condenser while one MSIV was partially open, and the sensitivity of Steam Generator levels to changes in steam demand. This report will be incorporated into Required Reading for licensed personnel to emphasize the interrelationship between these factors.

There was no damage to plant equipment or release of radioactivity as a result of this event, and at no time did conditions develop that may have posed a threat to the health or safety of the public.

There have been no previous similar occurrences.

KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

May 20, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Mr. E.H. Johnson, Director
Division of Reactor Safety and Projects
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 86-099
Re: Docket No. STN 50-482
Subj: Licensee Event Report 86-024-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Features actuation.

Yours very truly,

Glenn L. Koester

Glenn L. Koester
Vice President - Nuclear

GLK:see

Enclosure

cc: PO'Connor (2), w/a
JCummins, w/a

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