



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9  
License No. NPF-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated June 2, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specification as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-72 is hereby amended to read as follows:

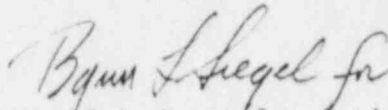
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(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 9 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective as of June 2, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION



Leif J. Norrholm, Acting Director  
Project Directorate III-2  
Division of Reactor Projects - III,  
IV, V and Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 10, 1988



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9  
License No. NPF-77

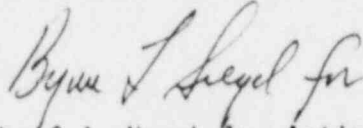
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated June 2, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specification as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 9 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. Attachment 2 contains revisions to Appendix A which are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective as of June 2, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION



Leif J. Norrholm, Acting Director  
Project Directorate III-2  
Division of Reactor Projects - III,  
IV, V and Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 10, 1988

ATTACHMENT TO LICENSE AMENDMENT NOS. 9 AND 9  
AND FACILITY OPERATING LICENSE NOS. NPF-72 AND NPF-77  
DOCKET NOS. STN-50-456 AND STN 50-457

Revise Appendix A as follows:

Remove Pages

3/4 3-9

3/4 3-12a

Insert Pages

3/4 3-9

3/4 3-12a

TABLE 4.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

| FUNCTIONAL UNIT                                     | CHANNEL CHECK | CHANNEL CALIBRATION                              | ANALOG CHANNEL OPERATIONAL TEST | TRIP ACTUATING DEVICE OPERATIONAL TEST | ACTUATION LOGIC TEST | MODES FOR WHICH SURVEILLANCE IS REQUIRED |
|---|---------------|--|---------------------------------|--|----------------------|--|
| 1. Manual Reactor Trip                              | N.A.          | N.A.   | N.A.                            | R                                      | N.A.                 | 1, 2, 3*, 4*, 5*                         |
| 2. Power Range, Neutron Flux                        |               |  |                                 |  |                      |  |
| a. High Setpoint                                    | S             | D(2, 4),<br>M(3, 4, 14)<br>Q(4, 6),<br>R(4, 5a)# | Q                               | N.A.                                   | N.A.                 | 1, 2                                     |
| b. Low Setpoint                                     | S             | R(4)#  | Q                               | N.A.                                   | N.A.                 | 1###, 2                                  |
| 3. Power Range, Neutron Flux,<br>High Positive Rate | N.A.          | R(4)#  | Q                               | N.A.                                   | N.A.                 | 1, 2                                     |
| 4. Power Range, Neutron Flux,<br>High Negative Rate | N.A.          | R(4)#  | Q                               | N.A.                                   | N.A.                 | 1, 2                                     |
| 5. Intermediate Range,<br>Neutron Flux              | S             | R(4, 5a)#  | Q                               | N.A.                                   | N.A.                 | 1###, 2                                  |
| 6. Source Range, Neutron Flux                       | S             | R(4, 5b, 12)#                                    | Q(9)                            | N.A.                                   | N.A.                 | 2##, 3, 4, 5                             |
| 7. Overtemperature $\Delta T$                       | S             | R(13)#   | Q                               | N.A.                                   | N.A.                 | 1, 2                                     |
| 8. Overpower $\Delta T$                             | S             | R#   | Q                               | N.A.                                   | N.A.                 | 1, 2                                     |
| 9. Pressurizer Pressure-Low<br>(Above P-7)          | S             | R#   | Q**                             | N.A.                                   | N.A.                 | 1  |
| 10. Pressurizer Pressure-High                       | S             | R#   | Q                               | N.A.                                   | N.A.                 | 1, 2                                     |
| 11. Pressurizer Water Level-High<br>(Above P-7)     | S             | R#   | Q                               | N.A.                                   | N.A.                 | 1  |

TABLE 4.3-1 (Continued)

TABLE NOTATIONS

- (12) At least once per 18 months during shutdown verify that on a simulated Boron Dilution Doubling test signal CVCS valves 112D and E open and 112B and C close within 30 seconds.
- (13) CHANNEL CALIBRATION shall include the RTD bypass loops flow rate.
- (14) On a one-time basis only, through June 15, 1988, the periodicity of this surveillance for Unit 1 only may be extended from 31 days (monthly) to 41 days.