PSNH

Public Service of New Hampshire

New Hampshire Yankee Division

SEABROOK STATION
Engineering Office

May 27, 1986

SBN- 1067
T.F. B7.1.2

United States Nuclear Regulatory Commission
Washington, DC 20555

Attention: Mr. Vincent S. Noonan, Project Director
PWR Project Directorate No. 5

References: (a) Construction Permits CPPR-135 and CPPR-136, Docket Nos. 50-443 and 50-444

Subject: Request for Additional Information: Deletion of the Boron Injection Tank

Dear Sir:

During discussions with the Staff on May 22, 1986, we were requested to provide additional information regarding the deletion of the boron injection tank and the supporting analysis. This additional information is provided in Attachment 1.

If you have any further questions, do not hesitate to contact us.

Very truly yours,

John DeVincentis
Director of Engineering

Enclosure

cc: Atomic Safety and Licensing Board Service List

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ATTACHMENT 1

Additional Information

1. The Boron Injection Tank (BIT), although physically located inside the Primary Auxiliary Building, is not connected to any system (see FSAR Figure 6.3-1, Sheet 2, provided in Amendment 56). That is, we do not rely on closed valves to isolate the BIT.

2. The analysis results in WCAP-9226 address two general categories of steamline break analyses.

   The first category addresses steamline breaks which occur while the reactor is critical and operating at power prior to reactor trip. Tube bundle uncovery is not predicted prior to reactor trip and the point of minimum DNBR during these transients and, therefore, there is no impact on the analysis results.

   The second category addresses steamline breaks which occur while the reactor is in hot zero power condition with the most reactive rod stuck out of the core. These transients are also analyzed on a plant specific basis in the Seabrook Station FSAR. The analysis methodology used in these analyses conservatively force a delay in the tube bundle uncovery well past the predicted tube bundle uncovery point.

   Therefore, a heat transfer rate is maintained in excess of the value which would be calculated with the combination of the predicted tube bundle uncovery and the superheated steam heat transfer. This is conservative since it maximizes the cooldown by the faulted steam generator and consequently maximizes the return to power. Therefore, the results presented in the Seabrook Station FSAR and WCAP-9226 remain valid.