

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) JAMES A. FITZPATRICK NUCLEAR POWER PLANT	DOCKET NUMBER (2) 0 5 0 0 0 3 3 3	PAGE (3) 1 OF 0 3
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TITLE (4)
Reactor Safety/Relief Valve Setpoint Drift

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																																																															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																																																													
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LICENSEE CONTACT FOR THIS LER (12)

NAME W. VERNE CHILDS, SENIOR LICENSING ENGINEER	TELEPHONE NUMBER
	AREA CODE: 3 1 5 TELEPHONE NUMBER: 3 4 9 - 6 3 0 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
B	AD	RV	T 0 2 0	Y	B	AD	RV	T 0 2 0	Y
B	AD	RV	T 0 2 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)
		MONTH: 0 9 DAY: 3 0 YEAR: 8 8

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

EIIS Codes are in []

During normal operation at 100% rated power on May 23, 1988, a vendor informed the plant staff that 3 of 5 Safety/Relief Valves (SRVs) [AD] that had been removed for test during a January 1988 maintenance outage did not actuate within the allowed +/-1% of nameplate setpoint as required by Technical Specification 2.2.1.B.

Setpoint drift and single valve inoperability has been previously evaluated and been acceptable with respect to overpressure protection. An evaluation for a previous event (LER-87-004), which is expected to also be bounding for this event, is in progress. The vendor test report with a complete evaluation of the apparent setpoint drift and corrective action taken is expected to be complete by August 1988.

Revision of this LER to reflect evaluation of potential overpressure and provide details concerning the event cause and corrective action is expected by September 30, 1988.

LER-87-004 and 85-009 are similar events involving SRV setpoint drift.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 365A's) (17)

EIIS Codes are in []

Description of Event

During a short maintenance and modification outage between January 9, 1988 and January 27, 1988, 5 of 11 Target Rock 2-Stage Safety/Relief Valves (SRVs) [AD] were removed for testing at a contractor facility. On May 23, 1988 the contractor informed the plant staff that 3 of the 5 SRVs being tested did not actuate within the +/-1% of nominal valve nameplate setpoint allowed by Technical Specification 2.2.1.B.

The actual actuations were as indicated below:

<u>Valve</u>	<u>Serial No.</u>	<u>Actuation Point in % of Nameplate</u>
02RV-71F	1097	106.4
02RV-71H	1111	102.5
02RV-71L	1062	107.5

Cause of Event

The cause of the apparent setpoint drift is not known at this time. However, previous similar events at this facility (see LER-87-004) and other facilities with Target Rock 2-Stage SRVs has revealed a generic problem of setpoint drift due to tolerance buildup in the pilot valve stem labyrinth seal area and due to pilot valve disc-to-seat bonding.

When the vendor completes testing, refurbishing and recertification of the valves, and returns them to the plant, a report is also provided which details the nature of problems and corrective action taken to provide reasonable assurance that the valves will actuate within +/-1% of nominal nameplate setpoint when reinstalled. This LER will be revised following receipt of the complete vendor report to provide additional information related to the cause of the event.

Analysis of Event

The remote actuation (operator demand) and Automatic Depressurization System (ADS) functions would not have been effected by the apparent setpoint drift.

Previous evaluations of the effects of SRV setpoint drift have demonstrated a large margin of pressure relief protection with one valve inoperable and other valves with setpoints above the nominal values allowed by Technical Specifications.

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TEXT (If more space is required, use additional NRC Form 386A's) (17)

An evaluation related to a previous similar event (see LER-87-004) is currently in progress. This evaluation is expected to define the most limiting (or bounding) cases of inoperable valves and also define the limiting setpoint drift conditions which are acceptable with respect to overpressure protection of the Reactor Coolant System [AD]. This LER will be revised following completion of the evaluation to provide additional information.

Corrective Action

No immediate corrective action was required because the valves had been replaced with recently refurbished and recertified valves prior to the end of the outage.

Long-term corrective action is to modify the valves by replacing the pilot valve discs with new discs using a different material. The modified valves will be installed during the next refueling outage (scheduled for fall 1988) as part of a test program for the Boiling Water Reactor Owners' Group (BWROG) SRV Setpoint Drift Committee. Participation in the BWROG SRV setpoint drift test program, which will monitor and compare performance of both modified and unmodified SRVs, will continue. If additional corrective action is indicated when the vendor report and overpressure evaluation discussed above are received, those actions will be included in the revised LER to be submitted following receipt of those reports.

Additional Information

Failed Component Identification:

- SRV Manufacturer: Target Rock Corp.
- Valve Model Number: 7567F
- Manufacturer NPRD Code: T020

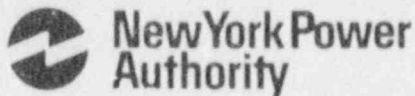
LER-87-004 and 85-009 are similar events which reported SRV setpoint drift.

Following receipt of vendor reports, this LER will be revised to provide the following additional information:

- cause of the apparent setpoint drift
- additional corrective action to reduce the recurrence of setpoint drift
- the effects of potential overpressure conditions due to valve setpoint drift or inoperability

Both reports are currently expected to be completed by August 1988. The revised LER is expected to be submitted by September 30, 1988.

James A. FitzPatrick
Nuclear Power Plant
PO Box 41
Lycoming, New York 13093
315 342 3840



Radford J. Converse
Resident Manager

June 13, 1988
JAFP-88-0533

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

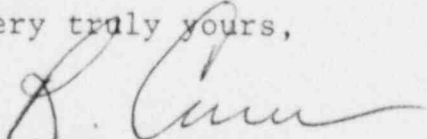
REFERENCE: DOCKET NO. 50-333
LICENSEE EVENT REPORT: 88-004-00

Dear Sir:

Enclosed please find referenced Licensee Event Report in accordance with 10 CFR 50.73.

If there are any questions concerning this report, please contact Mr. W. Verne Childs at (315) 349-6305.

Very truly yours,


RADFORD J. CONVERSE

RJC:WVC:lar

cc: USNRC, Region I (1)
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