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#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

FEB 1 7 1988

MEMORANDUM FOR: Hubert J. Miller, Director Division of Reactor Safety Region III

FROM:

Charles E. Rossi, Director Division of Operational Events Assessment Office of Nuclear Reactor Regulation

SUBJECT: TRANSFER OF OPERATING EVENT LONG-TERM FOLLOWUP

The Events Assessment Branch (EAE) has completed its short-term evaluation of an operating event at Brunswick 2 and has recommended certain long-term followup actions. These actions were based to a great extent on earlier followup by EAB on MSIV failures at Perry 1 for which Region III established an AIT. Copies of Event Followup Reports for Brunswick 2 and Perry 1 are enclosed. Because of the extensive work already done by your staff in evaluating operability of solenoid valves at Perry, we have concluded that additional longterm followup is warranted by your division. Thus, the purpose of this memorandum is to request your assistance to (1) confirm that the proposed activity in the Brunswick Report warrants your followup and (2) provide EAB with an expected completion date for completing that work.

We will monitor this activity until you inform us that your followup actions have been completed. To assist in this monitoring, the enclosed Technical Assignment Control (TAC) has been issued. We will extract data from the TAC/ RITS data base to provide a weekly status of long-term followup activities to headquarters and regional offices. Therefore, please establish a scheduled completion date and keep the TAC data current. This followup activity has been coordinated with R. Lanksbury of your staff.

The EAB contact for this effort is Jerry Carter; please direct any questions to him (x21194).

Charles E. Rossi, Director

Division of Operational Events Assessment Office of Nuclear Reactor Regulation

Enclosures: EFR 87-172 EFR 88-02 TAC 67092

- cc: E. Adensam
  - E. Sylvester
  - J. Carter
  - M. Reardon
  - R. Lanksbury

8806210128 880613 PDR FOIA MAXWELL88-165 PDR



### EVENT FOLLOWUP REPORT 87-172 50.72 EVENT #10515, OCTOBER 30, 1987 PLANT-PERRY 1 PROJECT MANAGER- T. COLBURN COGNIZANT ENGINEER- J. CARTER

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#### PROBLEM

MSIVs did not close within the time allowed by Technical Specifications.

#### CAUSE

Initially unknown but subsequently determined that a higher than anticipated ambient temperature caused the main pilot control (solenoid valve) to hangup.

#### SAFETY SIGNIFICANCE

Reactor isolation or containment integrity may not be possible in the event of an accident or transient.

#### DISCUSSION

During full closure tests of individual MSIVs, three valves exceeded the 5 second closure time of the technical specifications. Times were 22, 12, and 77 seconds with the two slowest valves being in the "D" steam line. Subsequent testing of these valves resulted in closure times of 3-5 seconds.

The licensee initially suspected that dirt in the air system caused the solenoid walve to hangup, thereby delaying the on-set of valve closure. Since there had been no previous instances of slow MSIV closure, the licensee continued power operation while reviewing maintenance records and preparing for their last pre-operational test, full MSIV isolation. The licensee and NRC regional and headquarters staff agreed with this approach.

Prior to running the full MSIV isolation test, the licensee again tested individual MSIVs for closure. Slow valve closure was observed again. The licensee shut down the reactor without performing their full MSIV isolation test.

NRC dispatched an AIT to the site to be present during disassembly of the solenoid values and to evaluate the problem. Initial conclusions were that steam leaks in the vicinity of the MSIVs and the control circuit values (includes the solenoid values) had caused ambient temperatures in the vicinity of 300° F at the highest temperature location which also was the location of the "D" isolation value. The elastomer seal in the solenoid value had hardened and was believed to have held the solenoid in the closed position thereby preventing air from being vented, thereby keeping the MSIV open.

#### FOLLOWUP

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The All will document their findings and present any followup to be done, plant specific or generic. No further action by EAB is necessary.

Jerty Carter BWR Section Events Assessment Branch

cc: T. Colburn E. Rossi

### FEB 1 0 1988

## BRUNSWICK 2 ISOLATION VALVES IN DRAIN LINES FAIL TO CLOSE

### EVENT FOLLOWUP REPORT 88-02 50.72 11109 JANUARY 2, 1988 PLANT-BRUNSWICK 2 PROJECT MANAGER-E. SYLVESTER COGNIZANT ENGINEER-J. CARTER

#### PROBLEM

Automatically operated isolation valves in both the equipment drain line and floor drain line from the drywell did not close upon demand.

#### CAUSE

Currently unknown; however, the solenoid valves venting air from the isolation valve pistons are suspected of sticking.

#### SAFETY SIGNIFICANCE

Two flow paths from primary containment to the environment existed after automatic isolation was required.

#### DISCUSSION

The facility was at 51% power and in the process of shutting down to refuel when a decreasing condenser vacuum led to a manual scram. Groups 2, 6 and 8 isolation on low level 1 were received as expected following the scram. All systems functioned as required except for the two automatic isolation valves in the drywell floor drain system (F-003 and F-004) and two automatic isolation valves in the drywell equipment drain system (F-019 and F-020). (See the attached schematic). These 4 valves did not close. The valves in the floor drain system were subsequently shut when a switch was placed in the closed position. The valves in the equipment drain line closed at different times after cycling the close switch. Subsequent to these closures, the valves were operated successfully when the drain sumps were pumped. An AIT was sent to the site to investigate the event.

Early investigation revealed that these valves use a single coil solenoid to vent and supply air to the piston which closes or opens the isolation valve. The solenoid valve design is different than the solenoid used in the control air system for MSIVs. Isolation valve failures in drain lines had been observed previously at Brunswick Unit 2; no failure of the corresponding valves at Unit 1 had been recorded. No known differences can explain why Unit 2 experienced failures. Disassembly of one solenoid valve believed to be sticking did not provide any apparent cause of failure or sticking.

#### FOLLOWUP

(1) The AlT documented the facts associated with their investigation of the event, and identified steps that the licensee could take to improve the plant specific problem. (AIT Report issued January 27, 1988).

(2) Sclenoid valve problems have been the cause of numerous isolation valve failures at many different facilities. As a result, many generic communications related to solenoid valve failures have been issued (most recently Circular 81-41, IN 85-17, IN 86-57 and INPO SER 57-85); yet, isolation valve failures attributed to solenoids continue to occur (Perry, LaSalle and Brunswick). For these reasons, the following additional long term actions should be taken:

- (a) Region III in conjunction with Region II should prepare, as they have been planning, an Information Notice which summarizes the problems with solenoids, what to look for during inspections of solenoids, and suggests precautions and surveillance that should improve solenoid reliability.
- (b) AEOD should determine the extent of Asco solenoid valve failures as indicated by a review of operating reactor data (LERs and NPRDS). Of particular interest are failures related to inadequacies of Asco solenoid valve design, air system quality inadequacies, and maintenance related deficiencies which might be indicative of inadequate training of technicians or poor technical information exchange between licensee: and the vendor. The data should be used to develop conclusions on failure frequency, causes of failures, safety systems/functions impacted by these failures and identification of plants affected. A safety assessment of the failures should also be performed. This safety assessment should evaluate plant safety degradations which have occurred in comparison to the design basis and should consider the risk significance of systems impacted, including the potential for common cause failure. This work should be completed and sent to NRR/EAB by early summer 1988.
- (c) AEOD should evaluate the results of the ASCO solenoid valve test program which was initiated by Cleveland Electric as a result of the numerous MSIV problems at Perry. This information, used in conjunction with the evaluation identified in (b) above, should help in the formulation of any longer term follow up activities or additional regulatory recommendations. At an appropriate point during their evaluation AEOD should consider contacting INPO, or other appropriate industry group, to obtain early industry involvement in taking action to improve solenoid valve reliability.

Jen Conter Derry Carter

BWR Section Events Assessment Branch

cc: E. Sylvester C. Rossi



Reportable Event number 11109 .

Facility : BRUNSWICK Unit : 2 Region : 2 Vendor : GE,GE Operations Officer : Ray Smith NRC Notified By : L. JOHNSON Rad Release : No Cause : Unknown Component : Date Notified : 01/02/88 Time Notified : 04:10 Date of Event : 01/02/88 Time of Event : 00:17 Classification : 10 CFR 50.72 Category 1 : SCRAM Category 2 : ESF Actuation Category 3 : Category 4 :

#### EVENT DESCRIPTION :

WHILE SHUTTING DOWN FOR A PLANNED OUTAGE, THE UNIT WAS MANUALLY SCRAMMED FROM 51% POWER DUE TO DECREASING CONDENSER VACUUM. IT IS SUSPECTED THAT THE DECREASING VACUUM WAS DUE TO EXISTING STEAM LEAKS BECOMING VACUUM LEAKS AS TURBINE LOAD WAS DECREASED. GROUPS 2, 6 AND 8 ISOLATIONS WERE RECEIVED FOLLOWING THE SCRAM AS EXPECTED ON LOW LEVEL 1. ALL SYSTEMS FUNCTIONED AS REQUIRED EXCEPT TWO DRYWELL FLOOR DRAIN CONTAINMENT ISOLATION VALVES (#2G16-F003 & F004) AND TWO DRYWELL EQUIPMENT DRAIN CONTAINMENT ISOLATION VALVES (#2G16-F019 & F020) DID NOT AUTOMATICALLY CLOSE AS EXPECTED. THE UNIT IS NOW IN COND 3 (HOT S/D) AND WILL BE PROCEEDING TO COND 4 (COLD S/D) FOR THE OUTAGE. THE NRC RESIDENT WILL BE NOTIFIED. (NOTIFIED R2 POTTER). \* \* \* UPDATE @ 1251 BY COOPER \* \* \* THE FOC & FOO4 VALVES CLOSED UPON RECEIPT OF A MANUAL CLOSE SIGNAL (SWITCH POSITION) BUT THE FO19 & FO20 VALVES CLOSED ONLY AFTER CYCLING THE VALVE SWITCH TO THE CLOSED POSITION SEVERAL TIMES. THE COPRESPONDING UNIT 1 VALVES (#1G16-F003, F004, F019 & F020) WERE TESTED & MANUALLY CLOSED (SWITCH POSITION) SATISFACTORILY. LICENSEE WILL TROUBLESHOOT / REPAIR THE UNIT 2 VALVES. WRITTEN REPORT TO FOLLOW. LICENSEE INFORMED RI. NOTIFIED R2DO POTTER.

January 4, 1966

PRELIMINARY NOTIFICATION OF EVENT OF UNUSUAL OCCURRENCE PNO-11-88-01

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. This information is as initially received without verification or evaluation, and is basically all that is known by the Region II staff on this date.

FACILITY:	Licensee Emergency Classification:
Carolina Power & Light company	Notification of Unusual Event
Brunswick Unit 2	Alert
Docket No. 50-324	Site Area Emergency
Southport, North Carolina	General Emergency
	X Not Applicable

SUBJECT: REGION II DISPATCHES AUGMENTED INSPECTION TEAM (AIT) TO BRUNSWICK

Following consultation with the Office of Nuclear Reactor Regulation (NRR) and the Office of Analysis and Evaluation of Operational Data (AEOD), Region II is dispatching an Augmented Inspection Team to Carolina Power and Light Company's Brunswick nuclear power plant, located near Southport, North Carolina, to make an in-depth determination of why primary containment isolation valves for the Unit 2 drywell equipment drains and drywell floor drains failed to automatically close after receipt of valid Group 2 isolation signal on January 2, 1988.

A manual scram had been initiated at 12:17 a.m. (EST) on January 2, when condenser vacuum reached 22" Hg (decreasing) during an planned shutdown for a refueling outage. Subsequently, a valid low level 1 signal (vessel level (162.5 inches) caused the group 2, 6 and 8 isolation signals. A total of four valves in both divisions failed to close as designed.

The two floor drains were subsequently closed from the control room, but the equipment drain values did not close. One equipment drain value was found closed approximately 5 minutes after the event, and the other at approximately 10 minutes after the event. During this time, the auto close signal for the value was sealed in.

The licensee cycled all four valves successfully later that morning when the crains had to be pumped. A licensee team has been investigating the event, but thus far no definitive cause has been found.

The NRC senior resident inspector responded to the site on January 3 and continues to monitor the licensee's activities, augmented by regional staff.

Unit 2 will be in a scheduled refueling and maintenance outage until April, 1988.

The State of North Carolina has been informed.

There was no injury to personnel, no release of radioactivity and no danger to public health and safety as a result of this event. The NRC's AIT is being dispatched to the site to provide an in-depth regulatory analysis of the event to assure that its cause is fully understood prior to restart of the unit.

This information is current as of 4:00 p.m. (EST) on January 4, 1988.

CONTACT\* P. FREDRICKSON - 242-5649

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January 4, 1988 MORNING REPORT - REGION II

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Resident Inspector HPCI Valve Motor Failure

Notification/Subject

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Event No N/A

On December 31 1987, with Unit 1 at full power, the HPCI steam admission valve meter tailed during routine surveillance testing. The Licensee mechanical problem was found with the valve or actuator. Cause of the motor tailure is unknown at this time. The licensee replaced the wolor tested the valve and declared the system operable on January 2.

The resident inspector will follow this event Regional Artion

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Not111cation/Subject

Duly Officer Mulii-Irain Containment Isolation Valve failure

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Event No. 11109

On Jarwary 7. 1986. primary containment isolation valves for the Unit 2 dryweill cruthment drains and dryweill floor drains (4 valves total both signal. A manual scram from 51% had been initiated at 12:17 a.m. on Jarwary 2. 1988. when condenser vacuum reached 227 Hq decreasing during signal investel level lass than 162.5 inclusible and from it isolation signals. The two floor drain valves were subsequently for coupment drain valve was found closed 5 inclusible and from the control room, but the equipment drain valves were subsequently for coupment drain valve was found closed 5 minutes alter the event and the other all 10 minutes after the event. During this time. The auto the other all to minutes after the drains when the drains and the invest-tions signal for the value monting when the drain valves were subsequently the license assembled a team to investing the drains had to be pumped to the license assembled a team to investing the drains had to be pumped is cruticalion for the investing when the drains had to be pumped to supret the last that morning when the drains had to be pumped to supret the last that morning the the fronts to date have included is fronted. The license has verified that the function of 18 month the roution function from wiring the function of 18 month is trutice. Unit 2 will be in an outage until April 1988.

The senior resident inspector responded to the site on January 3 and continues to monitor the licensee's activities. The resident staff will be augmented by

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regional specialisis to review this event

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Notification/Subject Region II Update of REN 11109

January 5. 1988

MORNING HEPORT - REGION II

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Evert No 11109

An humanish Investigation Team (AIT) has been dispatched to the Carolina Functions related to a multi-train failure of drywell equipment and licenser actions related to a multi-train failure of drywell equipment and licenser distincentialment isolation valves subsequent in a reactor scram of Unit 7 on January 2, 1998. At present, the licensee has not determined the islight methanism, but has been issued on this event operability on the

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Str./Hatch 2

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Notification/Subject Resident/Low Reactor Water Level

Event No N/A

Information: A low reactor water level occurred when the tecdwater controller failed high. The operator took manual control and restored the water level before the reactor scrammed. The water level reached a tevel of 12.5 inches as indicated by the SPDS system while the scram scitching is at 12.30 inches. One recorder in the Control Room indicated that the level was 10.5 inches. The license checked the scram found them to be properly set and functioning properly. The license is continuing to investigate this event and the Resident Inspector is attacted the investigate the

# CARCLINA POWER & LIGHT COMPANY

# BRUNSWICK STEAM ELECTRIC PLANT

## PLANT OPERATING MANUAL VOLUME XII

## UNIT O

PROCEDURE	TYPE:	CORRECTIVE MAINTENANCE INSTRUCTION (CM)
PROCEDURE	NUMBER:	0CM-5V004
PROCEDURE	TITLE:	ASCO 206-832 SERIES SOLENOID VALVES WITH RESILIENT SEATING

REVISION 1

APPROVED BY:

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General Manager/Manager - Maintenance

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8/11/87

Rev. 1

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0CM-5V004

FIGURE 1





DCM-SV004

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FIGURE 2

CATALOG 80173 EXPLOSION PROOF AND WATERTIGHT

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# TECHNICAL ASSIGNMENT CONTROL

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#### SECTION II. SYSTEMS CONTROL DATA

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solenoids, results of Perry testing and initiate "iscussions with industry.

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