ATTACHMENT 2

PROPOSED CHANGES

7.1 ORGANIZATION, REVIEW, AND AUDIT-ADMINISTRATIVE CONTROLS Applicability

Applies to the lines of authority and responsibility for the operational safety of the facility, and the organization for periodic review and audit of facility operation.

Objectives

To define the principal lines of authority and responsibility for providing continuing review, evaluation, and improvement of the plant operational safety.

SPECIFICATION AC 7.1.1 - ORGANIZATION, ADMINISTRATIVE CONTROLS

1. RESPONSIBILITY

- a. The Manager, Nuclear Production and Station Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.
- b. The Shift Supervisor (or during his absence from the control room and Shiit Supervisor's office, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Senior Vice President, Nuclear Operations shall be reissued to all station personnel on an annual basis.

3. ORGANIZATION

a. Onsite and Offsite Organizations

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- (1) Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels, including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR.
- (2) The Manager, Nuclear Production and Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

- (3) The Senior Vice President, Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- (4) The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

b. Unit Staff

- (1) Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 7.1-1.
- (2) A licensed operator must be in the control room at all times when fuel is in the reactor.
- (3) During shutdown and refueling conditions, an individual (other than the Technical Auvisor) with a valid RO (or SKO) license shall be present in the control room. During all other conditions, an individual (other than the Technical Advisor) with a valid SRO license shall be prosent in the control room.

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- (4) During the performance of reactor startup, reactor shutdown, and recovery from reactor trip, two licensed operators must be in the control room.
- (5) All cor- alterations shall be observed and ly supervised by either a licensed Senior nator Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- (6) An operator or technician qualified in radiation protection procedures shall be present at the facility at all times that there is fuel on site.
- (7) A site Fire Brigade of at least five members shall be maintained on site at all times*. The Fire Brigade shall not include 3 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.

^{*} Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

- (8) Members of the plant staff who perform safety related functions (e.g., Senior Reactor Operators, Reactor Operators, Auxiliary Operators, Health Physics Technicians, and key maintenance personnel) should, to the extent practicable, work an eight-hour day, 40-hour week, while the plant is operating. In the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown or refueling, major maintenance or major plant modifications, on a temporary basis, the following guidelines shall be followed:
 - (a) An individual should not be permitted to work more than 16 hours straight (excluding shift turnover time)
 - (b) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48hour period, nor more than 72 hours in any seven-day period (all excluding shift turnover time).
 - (c) A break of at least 8 hours should be allowed between work periods (including shift turnover time).

- (d) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on shift.
- (9) If unusual circumstances arise requiring deviation from the above guidelines, such deviation shall be authorized by the Manager, Nuclear Production and Station Manager, his designee, or higher levels of management, and with documentation of the basis for granting the deviation. Controls shall be established such that excessive individual overtime hours have not been assigned. The paramount consideration in overtime assignment shall be that significant reductions in the effectiveness of operating personnel would be highly unlikely. Routine deviation from the above guidelines is not authorized.

Authorized deviations to the working hour guidelines shall be documented and available for review by the Nuclear Regulatory Commission.

(10) The Shift Supervisors, and either the Operations
Manager or at least one Superintendent of
Operations shall hold a Senior Reactor
Operator's license. The Reactor Operators shall
hold a Roactor Operator's license.

- (11) Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 7.1-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members, "ovided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 7.1-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent. This provision also does not exclude the requirement for an SRO licensed individual to be in the control room at all times other than the shutdown conditions specified in Table 7.1-1.
- (12) During any absence of the Shift Supervisor from the control room or Shift Supervisor's office while the unit is in Hot, Cold, or Refueling Shutdown, an individual with a valid SRO license or RO license shall be designated to assume the control room command function. During any absence of the Shift Supervisor from the control room or Shift Supervisor's office during all other conditions, an individual (other than the Technical Advisor) with a valid SRO license shall be designated to assume the control room command function.

TABLE 7.1-1

INIMUM SHIFT CREW COMPOSITION

| Position | Number of Individuals Required to Fill Positio | |
|----------|--|----------------------|
| | During Hot. Cold. or Refueling Shutdown (a) | All Other Conditions |
| SS (SRO) | 1 | 1 |
| SRO | Not Required | 1 |
| RO | 1 | 2 (b) |
| EO | 1 | 1 |
| AT | Not Required | 1 |
| TA (c) | 1 | 1 |

SS - Shift Supervisor with a Senior Reactor Operator's License

SRO - Individual with a Senior Reactor Operator's License

RO - Individual with a Reactor Operator's License

EO - Equipment Operator AT - Auxiliary Tender

TA - Technical Advisor (on 1-hour call)

NOTES

- a. Per Technical Specification definitions, Section 2.0.
- b. One of the two Reactor Operators may be an Equipment Operator with a valid RO license provided that the staffing requirement for Equipment Operators is being met by another individual qualified as an Equipment Operator.
- c. The Technical Advisor shall be scheduled as required by Specification 7.1.1.3.b.(5).

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3. TECHNICAL ADVISORS

The function, responsibilities and authority of the Technical Advisors shall be as follows:

a. Function

The Technical Advisors shall function to make objective evaluations of plant conditions and to advise or assist plant management in correcting conditions that may compromise safety of operations.

b. Responsibilities

The Technical Advisors are responsible for:

- (1) Maximizing plant safety during and after accidents, transients, and emergencies by independently assessing plant conditions and by providing technical assistance to mitigate and minimize the effects of such incidents and make recommendations to the Operations Manager or the Superintendents of Operations.
- (2) Reviewing abnormal and emergency procedures,
- (3) Assisting the operations staff in applying the requirements of the Technical Specifications.
- (4) Providing evaluation of Licensee Event Reports from other plants as assigned, and

(5) Being in the control room within one hour after an emergency call. The Technical Advisors shall work on a normal day work schedule, but will be placed "on call" after normal working hours.

c. Authority

The Technical Advisors shall report to, and be directly responsible to, the Systems Engineering Manager. The Technical Advisors shall maintain independence from normal plant operations to be objective in their evaluations.

4. UNIT STAFF QUALIFICATIONS

- a. The staifing of the plant shall be in accordance with American National Standards Institute (ANSI) N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants".
- b. Each member of the facility staff shall ~ t or exceed the minimum qualifications of ANSI N18.1-1971 for the comparable position, except for the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September, 1975.

5. TRAINING

- a. A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR 55.
- b. A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training Manager and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training/drill sessions which shall be held at least once per calendar quarter.
- c. The Technical Advisors shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents. An initial training and retraining program for the Technical Advisors shall be maintained under the direction of the Nuclear Training Manager. The Technical Advisors shall also receive training in plant design and layout, including the capabilities of instrumentation and controls in the control som.

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Specification AC 7.1.2 - Plant Operations Review Committee (PORC), Administrative Controls

The organization, responsibilities, and authority of the PORC shall be as follows:

1. Membership

The Plant Operations Review Committee shall be composed of the following:

Chairman: Manager, Nuclear Production and Station
Manager
Nuclear Support Manager
Superintendent of Chemistry and Radiation Protection
(Radiation Protection Manager)
Superintendent of Operations
Maintenance Department Manager
Systems Engineering Manager
Superintendent of I&C Maintenance
Operations Manager
Nuclear Training Manager

2. Alternates

An alternate chairman and alternate members, if required, shall be pointed in writing by the PORC Chairman to serve in the absence of a chairman or a member; however, no more than two alternate members shall participate as voting members in PORC activities at any one time.

3. Meeting Frequency

The PORC shall meet at least once per calendar month and as convened by the Chairman or his designated alternate.

4. Quorum

A quorum shall consist of the Chairman or alternate Chairman, and four members including alternates.

5. Responsibilities

The PORC shall be responsible for:

- a. Review of all procedures required by Technical Specification 7.4(a), (b), (c), (d) and changes thereto, and any other proposed procedure or changes to approved procedures as determined by the Manager, Nuclear Production and Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering the evaluation and recommendations to prevent recurrence to the Manager, Nuclear Production and Station Manager and to the Chairman of the Nuclear Facility Safety Committee.
- f. Review of all Reportable Events.
- Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations, and reports thereon as requested by the Chairman of the Nuclear Facility Safety Committee.
- Review of the Plant Security Plan and implementing procedures.
- j. Review of the plant Radiological Emergency Response Plan and implementing procedures.

6. Authority

The PORC shall:

- a. Function to advise the Manager, Nuclear Production and Station Manager on all matters that affect nuclear safety.
- b. Recommend to the Manager, Nuclear Production and Station Manager in writing, approval or disapproval of items considered under 5.a through 5.d, above.
- c. Render determinations in writing with regard to whether or not each item considered under 5.a through 5.e above constitutes an unreviewed safety question.
- d. Provide immediate written notification to the Manager, Nuclear Production and Station Manager, and the Chairman of NFSC of disagreement between the PORC and the Manager, Nuclear Production and Station Manager; however, the Manager, Nuclear Production and Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.a above.

7. Records

The PORC shall maintain written minutes of each meeting and copies shall be provided to the Manager, Nuclear Production and Station Manager, and Chairman of the Fort St. Vrain Nuclear Facility Safety Committee.

Specification AC 7.1.3 - Nuclear Facility Safety Committee (NFSC), Administrative Controls

The organization, responsibilities, and authority of the NFSC shall be as follows:

1. Function

The Nuclear Facility Safety Committee shall collectively have the competence required to review problems in the following areas:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radiochemistry
- d. Metallurgy

- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical and Electrical Engineering
- h. Quality Assurance Practices
- Other appropriate fields associated with the unique characteristics of the nuclear power plant.

2. Membership

The NFSC shall be composed of the following:

Chairman (As appointed - See Step 4)

Manager, Nuclear Licensing and Resource Management Division

Manager, Nuclear Production and Station Manager

Manager, Nuclear Engineering Division

Manager, Quality Assurance Division

Safety and Security Director

Consultants, as required, shall be appointed in writing by the Senior Vice President, Nuclear Operations.

3. Alternates

Alternate members, if required, shall be appointed in writing by the Senior Vice President, Nuclear Operations; however, no more than two alternate members shall participate as voting members in NFSC activities at any one time.

4. Chairman

The Chairman and Alternate Chairman of the NFSC shall be appointed in writing by the Senior Vice President, Nuclear Operations and shall serve as members of the NFSC.

5. Consultants

Consultants shall be utilized as determined by the Chairman, NFSC, to provide expert advice to the NFSC.

6. Meeting Frequency

The NFSC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

7. Quorum

A quorum of the NFSC shall consist of the Chairman or his designated alternate and a majority of the NFSC members including alternates. No more than a minority of the quorum shall have line responsibilities for operation of the facility.

8. Responsibilities

- a. The Nuclear Facility Safety Committee shall review:
 - (1) The safety evaluations for safety significant changes to procedures, equipment, or systems

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personnel or an outside fire protection firm;

- (b) a biennial audit of the fire protection program and implementing procedures;
- (c) a triennial fire protection and loss prevention inspection and audit utilizing an outside qualified fire consultant.
- (9) The Offsite Dose Calculation Manual and Process Control Program and implementing procedures at least once per 24 months.
- (10) The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months.
- (11) The performance of activities required by the Quality Assurance Program to meet the provisions of Regulatory Guide 1.21 Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975, at least once per 12 months.

9. Authority

The NFSC shall report to and advise the Senior Vice President, Nuclear Operations on those areas of responsibility specified in 8.a, 8.b and 8.c above.

10. Records

Records of NFSC activities shall be prepared, approved, and distributed as indicated below:

a. Minutes of each NFSC meeting shall be prepared and forwarded to the Senior Vice President, Nuclear Operations within 30 days following each meeting.

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- b. After preliminary approval by the Senior Vice President, Nuclear Operations, the minutes shall be distributed to all NFSC members, and approved at the next NFSC meeting.
- c. Reports of reviews encompassed by Section 8.a, above shall be forwarded to the Senior Vice President, Nuclear Operations, within 30 days following completion of the review.
- d. Audit reports encompassed by Section 8.c. above shall be forwarded to the Senior Vice President, Nuclear Operations, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

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7.2 SAFETY LIMITS, ADMINISTRATIVE CONTROLS

Applicatility

Applies to the administrative procedures to be followed in the event that a safety limit is exceeded.

Objectives

To define the administrative procedures which will be followed in the event that a safety limit is exceeded.

Specification AC 7.2 - Action to be Taken if a Safety Limit is Exceeded, Administrative Controls

If a safety limit is exceeded, as defined in Specification SL 3.1 and 3.2, the following action shall be taken:

- a. The reactor will be shut down immediately and reactor operations shall not be resumed until approval is received from the NRC.
- b. The safety limit violation shall be reported to the Commission, the Manager, Nuclear Production and Station Manager, and to the Chairman, NFSC immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Chairman, NFSC and the Manager, Nuclear Production and Station Manager within ten days of the violation.

ATTACHMENT 3

SIGNIFICANT HAZARDS CONSIDERATION

SIGNIFICANT HAZARDS CONSIDERATION

I. EVALUATION OF CHANGES

A. Specification AC 7.1.1 is revised as follows:

Section 1. "RESPONSIBILITY," was added delegating control room command responsibility and corporate responsibility for overall nuclear plant safety. This section directly correlates to Section 6.1 of the Westinghouse Standard Technical Specifications (STS), NUREG-0452.

Section 2.a, "Onsite and Offsite Organization," was added to provide a directive for the establishment and definition of lines of authority, responsibility, and communication for onsite and offsite organizations. This section directly correlates to Section 6.2.1 of the STS, as amended by Generic Letter 88-06 for deletion of the organization charts from the Technical Specifications (Tech. Specs.).

Section 2.b(1) designates on-duty shift minimum shift crew composition per Table 7.1-1. This section follows the guidelines of Generic Letter 88-06, and incorporates the requirements of current Section 2.a.

Sections 2.b(2,3,4) discuss licensed operator on-shift requirements. These sections are added to conform to the formatting effort of this amendment; and include part of the current clarification text following Table 7.1-1 and the requirements of current Section 2.b.

Sections 2.b(5,6,7,8,9) are reformatting and editorial corrections, which incorporate the requirements of current Sections 2.c, 2.d, 2.e, 2.i, and the position titles of the Fort St. Vrain (FSV) reorganization.

Section 2.b(10) was added to delineate those requiring a Senior Reactor Operator's (SRO) license and those requiring a Reactor Operator's (RO) license; and follows the guidelines of Generic Letter 88-06.

Section 2.b(11) discusses shift crew composition and incorporates the requirements of the current final clarification paragraph following Table 7.1-1, which were not included in proposed Sections 2.b(2,3,4).

Section 2.b(12) was added to delineate control room command responsiblity in the absence of the Shift Supervisor, and further expounds on proposed Section 1.b.

Table 7.1-1 was relocated within Specification AC 7.1.1. The table retains the same requirements as the current Table 7.1-1. However, the page is reformatted to include the applicable notes.

Section 3, "TECHNICAL ADVISORS," is a reformatting addition. This section incorporates the requirements of current Sections 1.c and 2.f, with position titles per the FSV reorganization.

Section 4, "UNIT STAFF QUALIFICATIONS," is a reformatting addition and incorporates the requirements of current Sections 2.g and 2.h. "Upon commencement of commercial operation" is deleted since this stipulation is not necessary.

Section 5, "TRAINING," reformats and incorporates the requirements of current Section 3. "Compliance with Section 5.5 of ANSI 18.1-1971 shall be chieved no later than 6 months following commencement of commercial operation" is deleted since this stipulation is not necessary.

B. Specification AC 7.1.2 is revised as follows:

Section 1, Plant Operations Review Committee (PORC) Membership, is revised to incorporate position titles of the FSV reorganization. No expertise is deleted from the PORC, and the positions meet the description of ANSI N18.1.

Sections 5.a, 5.e, 6.d and 7 contain a position title change only. Responsibility and expertise remain the same.

Sections 3, 4, 9, and 10 contain position title change only. Responsibility and expertise remain the same.

Section 5.k, relative to PORC review of every unplanned onsite release of radioactive material to the environs, is deleted. This requirement is considered to be adequately covered in Sections 5.a through 5.g. Also, several recent plant Technical Specifications have been found not to include this requirement: River Bend 1, Grand Gulf 1, Nine Mile Point 2, and Palo Verde 1. Deletion of Section 5.k is also consistent with the Tech. Spec. Upgrade Program draft.

C. Specification AC 7.1.3 is revised as follows:

Section 1 contains only formatting changes.

Section 2 contains revisions to the Nuclear Facility Safety Committee (NFSC) Membership relative to position titles per the FSV reorganization. The actual membership and areas of responsibility/expertise remain the same.

Sections 3, 4, and 9 contain a position title change only. Responsibility and expertise remain the same.

Section 10.a is revised to delete the requirement that each NFSC meeting's minutes be approved within 30 days following each meeting. Section 10.b was added to direct the preliminary approval of the NFSC meeting minutes by the Senior Vice President, Nuclear Operations. It also directs the distribution of the minutes to the NFSC members, and approval of the minutes at the next NFSC meeting. These revisions will ensure that the entire NFSC membership will be given the opportunity to vote on the approval of the minutes of the last NFSC meeting.

D. Pages 7.1-20, 7.1-21, 7.1-22, and 7.1-23 are deleted:

Pages 7.1-20 and 7.1-21 contain Table 7.1-1 and its associated notes and clarification information. All this information is included elsewhere in the proposed amendment.

Pages 7.1-22 and 7.1.23 contain the organization charts (Figures 7.1-1 and 7.1-2). These charts are deleted from the Tech. Specs., based on the recommendation of Generic Letter 88-06.

E. Specification AC 7.2 is revised as follows:

Sections b. and d. contain a position title change only. Responsibility and expertise remain the same.

II. CONCLUSION

Except as otherwise noted above, this proposed amendment reformats current Administrative Controls Specification 7.1.1 requirements to better conform to STS formatting; deletes organization charts per the guidelines of Generic Letter 88-06; and retitles certain positions in AC 7.1.1, 7.1.2, 7.1.3. and 7.2 to conform to the FSV reorganization begun May 12, 1988.

Based, on the above, this proposed change does not involve a significant hazards consideration because operation of FSV in accordance with this change would not: involve a significant increase in the probability or consequences of an accident previously evaluated; create the possibility of a new or different kind of accident from any accident previously evaluated; involve a significant reduction in a margin of safety.