



Duquesne Light

Beaver Valley No. 2 Unit Project Organization
SEG Building
P.O. Box 328
Shippingport, PA 15077

2NRC-6-059

(412) 643-5200

Telecopy (412) 643-5200

Ext. 149

June 6, 1986

United States Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Mr. Peter Tam, Project Manager
Division of PWR Licensing - A
Office of Nuclear Reactor Regulations

TAC
62908

SUBJECT: Beaver Valley Power Station - Unit No. 2
Docket No. 50-412
SER Confirmatory Item 25,
Containment Sump 50% Blockage Assumption

REFERENCES: 1) NUREG-1057
2) NUREG-0897, Rev. 1
3) NUREG-0869, Rev. 1

The BVPS-2 Safety Evaluation Report (SER), Reference 1, confirmatory item 25, recommends that Duquesne Light Company (DLC) provide a debris generation and transport analysis to justify the acceptability of the Regulatory Guide (R.G.) 1.82 (Rev. 0) assumption of 50% sump blockage after a LOCA. Earlier responses to FSAR questions 480.2, 480.26 and 730.1 indicate that a minimum amount of fibrous insulation is employed within that containment and that a blockage of 50% of the flow area is assumed. FSAR Section 1.8 indicates BVPS-2 meets the intent of the R.G. 1.82 (Rev. 0) and FSAR Section 1.9 indicates that BVPS-2 conforms with Standard Review Plan (SRP) Section 6.2.2 (Rev. 3).

Following the issue of the BVPS-2 SER, Generic Letter 85-22 was issued. It states that this subject has been addressed in the efforts to resolve Unresolved Safety Issue A-43 and it refers to the documented technical findings, Reference 2. The Generic Letter states that, as a result of these efforts, the staff has issued R.G. 1.82 (Rev. 1) and SRP Section 6.2.2 (Rev. 4) which replace the 50% blockage assumption with a plant-specific debris evaluation. However, the Generic Letter notes that the staff's regulatory analysis, Reference 3, does not support the application of R.G. 1.82 (Rev. 1) to any plant now licensed to operate or that is under construction.

As recommended in the Generic Letter, if an increase in the fibrous insulation on primary coolant system piping and components inside containment is considered after BVPS-2 becomes operational, its safety significance on potential sump blockage will be evaluated and reported in accordance with 10 CFR 50.59 and will appropriately consider available NRC guidance at that time.

IFCA Add: Peter Tam 1/0

United States Nuclear Regulatory Commission
Mr. Peter Tam, Project Manager
SER Confirmatory Item 25,
Containment Sump 50% Blockage Assumption
Page 2

Based on the foregoing, DLC requests notification by July 31, 1986
that this SER confirmatory item is closed.

DUQUESNE LIGHT COMPANY

By RE Martin for
J. J. Carey
Vice President

JJS/kam

cc: Mr. P. Tam, Project Manager
Mr. L. Prividy NRC Resident Inspector
Mr. W. Troskoski, Sr. Resident Inspector