

Docket 50-285



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001
November 6, 1998

Mr. S. K. Gambhir
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 399
Hwy. 75 - North of Fort Calhoun
Fort Calhoun, Nebraska 68023-0399

SUBJECT: FORT CALHOUN STATION (FCS), UNIT NO. 1, RELIEF FROM ASME BOILER AND PRESSURE VESSEL CODE SECTION III, OVERPRESSURE PROTECTION REQUIREMENTS (TAC NO. M95802)

Dear Mr. Gambhir:

By letter dated May 21, 1996, Omaha Public Power District (OPPD) requested permanent relief from the requirements of Article 9, Paragraph N-910.8 of the 1968 Edition of the ASME Code, Section III. The requirements pertain to the spent fuel pool and shut down heat exchangers which were constructed as Class C vessels per the 1968 Edition of the ASME Code, Section III. The relief would allow OPPD to permanently retain the installation of a locked-open manual isolation valve (AC-140) located in series with overpressure protection relief devices in the chemical and volume control system and use specific administrative controls to verify its position.

In its May 21, 1996, letter, OPPD requested that the NRC staff authorize the installed valve configuration at FCS as an alternative to the above ASME Code design requirements pursuant to 10 CFR 50.55a(a)(3). OPPD stated that the manual isolation valve and relief valve configuration provides an acceptable level of quality and safety because administrative controls (AC-10 is chained, locked in the open position, and verified) are in place to ensure that the manual isolation valve will remain in the open position. OPPD also indicated that compliance with the above Code requirements would result in difficulties without a compensating increase in the level of quality and safety.

1/0

The NRC staff has reviewed OPPD's request for the NRC staff to authorize its proposed alternative pursuant to 10 CFR 50.55a(a)(3). The NRC staff finds that ASME Code, Section III requirements for Quality Group C components (ASME Code Class 3 components) such as the spent fuel pool and shut down heat exchangers, as stated in 10 CFR 50.55a(e), apply to nuclear power plants whose applications for construction permits (CPs) were docketed after May 14, 1984. The CP for FCS was docketed prior to May 14, 1984; therefore, the regulations in 10 CFR 50.55a(e) concerning ASME Code, Section III design requirements for Class 3 components do not apply to FCS. Accordingly, authorization of an alternative to ASME Code, Section III design requirements pursuant to 10 CFR 50.55a(a)(3) is unnecessary and inappropriate for FCS.

d/f/b

9811100258 981106
PDR ADCK 05000285
P PDR

103009

NRC FILE CENTER COPY

In its Updated Safety Analysis Report, GPPD committed to design and construct FCS in accordance with ASME Code, Section III design provisions. In its May 21, 1996, letter, OPPD identified isolation valves installed in FCS that did not meet the design requirements of ASME Code for a Class 3 (or Class C) component. The NRC staff views the licensee's discovered condition of non-compliance with the ASME Code, Section III design provisions as a deviation from plant licensing commitments. Accordingly, OPPD may either modify the plant to conform to the provisions of the current licensing and design basis information, or change the current licensing and design basis information to accurately reflect the existing plant design. Action may be taken pursuant to the provisions of 10 CFR 50.59, and depending on the results of OPPD's evaluation, a license amendment may be required.

On the basis of the above evaluation, the NRC staff concludes that authorization of an alternative or relief from ASME Code, Section III design provisions for ASME Code Class 3 components is unnecessary and inappropriate for FCS. OPPD may resolve the existing deviation by modifying the plant or by revising the licensing and design information.

This completes our review for TAC No. M95802. Please contact L. Raynard Wharton at (301) 415-1396 if you have any questions regarding this issue.

Sincerely,

Original Signed By

William H. Bateman, Director
 Project Directorate IV-2
 Division of Reactor Projects - III/IV
 Office of Nuclear Reactor Regulation

Docket No. 50-285

cc: See next page

DISTRIBUTION:

Docket File EPeyton
 PUBLIC OGC
 PDIV-2 Reading ACRS
 EAdensam (EAG1) RWessman
 WBateman PGwynn
 RWharton GHill (2)
 DLange, EDO

DOCUMENT NAME: FC95802.LTR

OFC	PDIV-2/PM	PDIV-2/LA	EMEB/C	OGC <i>W/NLO</i>	PDIV-2/D
NAME	<i>R</i> Wharton	<i>esp</i> EPeyton	<i>W</i> RWessman	<i>M</i> Young	<i>W</i> WBateman
DATE	10/13/98	10/13/98	10/14/98	10/26/98	10/5/98

OFFICIAL RECORD COPY

cc:

Winston & Strawn
ATTN: Perry D. Robinson, Esq.
1400 L Street, N.W.
Washington, DC 20005-3502

Mr. Jack Jensen, Chairman
Washington County Board
of Supervisors
Blair, Nebraska 68008

Mr. Wayne Walker, Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 309
Fort Calhoun, Nebraska 68023

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Ms. Cheryl Rodgers, LLRW Program Manager
Environmental Protection Section
Nebraska Department of Health
301 Centennial Mall, South
P.O. Box 95007
Lincoln, Nebraska 68509-5007

Mr. J. M. Solymossy
Manager - Fort Calhoun Station
Omaha Public Power District
Fort Calhoun Station FC-1-1 Plant
Post Office Box 399
Hwy. 75 - North of Fort Calhoun
Fort Calhoun, Nebraska 68023

Mr. Mark T. Frans
Manager - Nuclear Licensing
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 399
Hwy. 75 - North of Fort Calhoun
Fort Calhoun, Nebraska 68023-0399