

CLINTON POWER STATION
10CFR50.59 REPORT
FOR
PLANT MODIFICATIONS
TEMPORARY MODIFICATIONS
PROCEDURES AND DOCUMENTS

CLINTON POWER STATION
10CFR50.59 REPORT
FOR
MODIFICATIONS
FROM JANUARY THROUGH MARCH
1988

10CFR50.59 REPORT FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0070 FA AF023

INSTALL FENCE IN RADWASTE BUILDING AND SCREENHOUSE

This plant modification installs a fence in the radwaste building machine shop. This fence will function as a Radiological Control Area boundary. This modification also provides a fenced tool area in the screenhouse and in the radwaste building for the safe keeping of tools. Collapse of these fences will not affect any equipment important to safety because the fences are located in non-safety, non-seismic areas.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0100 SM AR-33 SUPP 1 ELIMINATE ALARM RELAY FEEDBACK SIGNAL IN RADIATION MONITORS

This modification supplement disconnects the alarm relay feedback signal from the microprocessor in each of the area radiation/process radiation (AR/PR) monitors which occurs during the source check. This modification also corrects the AR system firmware to prevent the display of incorrect data during channel source checks. Eliminating the alarm circuitry feedback signal during the channel source check sequence is acceptable because proper functioning of the alarm circuitry is verified by the channel functional test.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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NUMBER EVALUATED

TITLE

88-0009 PM AR-34

MODIFY RADIATION CENTRAL CONTROL TERMINALS

This plant modification changes the firmware on the area radiation/process radiation (AR/PR) system central control terminals (CCTs). These changes will correct errors that cause the CCTs to fail and will add a continuous audible alarm for FAIL and ALERT lighted alarms. This plant modification also changes the FAIL and ALERT tiles from stationary lights to lighted pushbutton alarm acknowledge switches. This modification enhances the reliability of main control room annunciators and increases the operator's awareness of significant plant conditions.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0018 FA ASF004

ABANDON ELECTRODE BOILER RECIRCULATION FLOW MONITORS

Differential pressure switches OPDS-AS023 and 24 monitor auxiliary steam system electrode boiler recirculation flow by sensing differential pressure across recirculation pumps OAS05PA and B. This field alteration abandons the differential pressure switches in place to eliminate unnecessary electrode boiler trips caused by the switches. This change will permit electrode boiler operation without recirculation flow. This will not damage the boiler, but will result in lowered steam output. Overall auxiliary steam system reliability will be improved.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG DOCUMENT
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TITLE

88-0083 FA CDF005

DELETE DIFFERENTIAL PRESSURE ALARMS ACROSS CONDENSATE PUMP

This field alteration eliminates the high differential pressure alarms 1PDS-CD039, 040, 041, and 042, across the condensate pump suction strainers. Upon completion of power ascension testing, the fine mesh screens from the temporary in-line strainers were removed from all condensate pumps. However, the high differential pressure alarms which remain erroneously annunciate. Clogging of coarse mesh screens which are still in place will be identified through daily monitoring of local pressure gauges.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

87-2654 FA CF009

REVISE DATA SHEETS FOR TIME DELAY RELAYS

This field alteration corrects discrepancies between nameplate data and seven data sheets and adds a coil rating for Agastat time delay relays located in the main control room. The discrepancies consist of model numbers, purchase part drawing numbers, and instrument service and qualification descriptions. The corrections to the model numbers and purchase part drawing numbers also need to be made on FSAR Table 3.10-2. These time delay relays are part of the diesel generator room ventilation, standby gas treatment, suppression pool makeup, fuel pool cooling and cleanup, turbine generator, turbine generator electro-hydraulic control, containment building ventilation, and the fuel building ventilation systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG DOCUMENT
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TITLE

88-0078 FA CWF005

REVISE DRAWING TO SHOW EXPANSION JOINTS

The piping and instrumentation diagram (P&ID) for the circulating water (CW) system (FSAR Figure 10.4-3) shows components 1CW02MA and B as restriction devices. This field alteration revises the P&ID to correctly show the components as expansion joints. Expansion joints 1CW02MA and B were installed as part of the original design to reduce the effects of thermal stress from the main condenser on the waterbox drain lines which are fixed in the basement floor of the turbine building.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0023 PM FP-80

PROVIDE A STORAGE / MAINTENANCE AREA IN THE RADWASTE BUILDING

This plant modification installs a welded wire mesh fence with a gate and two fire extinguishers in the cable tunnel at elevation 781' of the radwaste building. This fence will change the cable tunnel into a storage and maintenance area to relieve crowded conditions in the control and instrumentation (C&I) shop. Use of this area for storage and maintenance will not affect the structural integrity of the radwaste building or the existing ventilation system. The added fire extinguishers provide the required fire protection. Also, there is no safety related cable or permanent plant equipment in the area.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0035 FA GSF003

REVISE POSITION OF GLAND SEAL SYSTEM VALVES

The gland steam system seal steam packing exhauster/condensers are designed to draw air and steam from the main turbine shaft packings. This field alteration revises the piping and instrumentation diagram (FSAR Figure 10.4-2) to show the steam packing exhauster (SPE) drain controls and loop seal fill valves as normally open. This will allow operations personnel to field adjust the valves so that the SPE drain/loop seal can be maintained at the proper level. This change will prevent the condenser from experiencing excessive air in-leakage. The gland seal system is not safety related and these changes will not impact systems that perform a safety function.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0085 FA HOF007

REMOVE HEATER DRAIN CHECK VALVE 1HD002B

This field alteration removes heater drain check valve 1HD002B from the normal drain line of the moisture separator drain tank and replaces it with a pipe section. This check valve was provided to prevent a backflow of flashing water and steam from interfering with tank drainage and causing a turbine trip during turbine load reductions. Analysis showed that routine load reductions would not interfere with tank drainage if the check valve were removed. Also, unanticipated load reductions of sufficient magnitude to affect tank drainage would only be caused by equipment failures which would have tripped the turbine themselves.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0033 FA WPF003

CHANGE MATERIAL OF HP SYSTEM PUMP TOP SHAFT

This field alteration allows a part substitution for the high pressure core spray (HP) system pump top shaft. The original top shaft material met the specification of American Society for Testing and Materials (ASTM) Standard A-276. The spare replacement shaft meets the specification of ASTM A-479. This material substitution is acceptable because the physical properties of the replacement materials, i.e., tensile strength, yield strength, and elongation, are equivalent or better than the original material.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0037 PM LD-22

INSTALL JUMPERS TO BYPASS LEAK DETECTION SYSTEM HANDSWITCHES

Leak detection system handswitches RE001, RE002, RF008 and RF009 in panel 1H13-P601 in the Main Control Room are not wired in accordance with the design. The present configuration renders the containment equipment and drywell area sump fill timers inoperable. This plant modification installs jumpers which will bypass and eliminate the need for the handswitches and provide power to the sump fill timers. The function of the handswitch will be performed by the remaining reset switch. This modification will allow the leak detection system to operate as originally intended.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.55 REPORT FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0008 PM M-56

INSTALL FLUSHING LINE TO FLOOR AND EQUIPMENT DRAIN LINES

This plant modification installs a flushing line from the makeup condensate (MC) system to the acid/caustic drain lines of the equipment drain (WE) system and the floor drain (TF) system. This line will allow flushing of the drain lines to prevent plugging following acid/caustic additions. This modification also adds lined check valve at acid-water interfaces to prevent corrosion, and alters the slope of WE/TF drain lines from the WE acid/caustic system. This modification does not affect safety related, seismic, or augmented D components. The affected portions of the WE and TF systems are not used for the handling of radioactive fluids.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0074 FA MF014

ALLOW USE OF TEMPORARY STRAINERS IN RADWASTE SYSTEMS

This field alteration adds a note to piping and instrumentation diagrams (FSAR Figures 11.2-2 and 11.4-3) to allow the use of temporary strainers in radwaste systems at times other than during preoperational testing and flushing. This will allow temporary strainers to be added, as necessary, to prevent resins from entering unwanted areas. Failure analysis of the strainers is not required to meet ANSI B31.1, the design for this class D piping system. Also, the strainers will not impact pump operation because the original design accounted for the strainers and the net positive suction head difference they cause.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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NUMBER EVALUATED

TITLE

88-0003 PM PR-34

REMOVE DOWNSCALE & POWER FAIL TRIP FROM PR MONITORS

This plant modification removes the downscale and power fail trip functions from 16 process radiation (PR) monitors and adds handswitches to allow a manual trip signal to be initiated. The 16 monitors detect radioactivity in the containment fuel transfer pool vent plenum, in the exhaust ducts of the containment building ventilation system, in the fuel building ventilation system, and in the continuous containment purge system. The trip outputs of these monitors isolate the containment building ventilation system and the fuel building ventilation system, and start the standby gas treatment system. Downscale and power fail annunciation for these monitors is provided in the main control room. This provision will ensure the manual trip of a failed channel within the time limit specified by the Technical Specifications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0022 FA PRF004

REMOVE FILTERS IN LIQUID PROCESS RADIATION MONITORS

This field alteration removes filters from the inlets to seven off-line liquid process radiation (PR) monitors. These monitors measure the radioactivity in the liquid radwaste discharge, in the shutdown service water effluent, in the fuel pool heat exchanger service water, in the plant service water effluent, and in the component cooling water effluent. The filter is not part of the vendor's standard design and provides an unnecessary flow restriction. Removal of the filters will improve the operating characteristics and accuracy of the PR monitors. This modification will not adversely affect radioactive releases or other systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

09/27/88

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NUMBER EVALUATED

TITLE

88-0013 PM PS-13

ADD SAMPLE LINE ISOLATION BLOCK VALVES

This plant modification installs sample line isolation block valves to each condensate demineralizer sampling station. This will allow the isolation of sampling stations for maintenance purposes. The sample lines affected by this modification are not safety related or seismically qualified.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

87-2673 FA RNF001

DELETE AUTOMATIC RUNBACK OF RR FLOW CONTROL VALVES

This field alteration deletes the automatic runback of the reactor recirculation (RR) flow control valves. Automatic runback of these valves was initiated by a circulating water pump trip when all three pumps were operating with a low condenser vacuum. This is a power generation feature. Loss of condenser vacuum transients described in FSAR Chapter 13 do not take credit for this runback feature.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0017 FA FAF003

REVISE SAFETY CLASSIFICATION OF VALVES 1SX095A AND B

This field alteration revises the safety classification of spare valves 1SX095A and B from active to passive and from 1E power to non-1E power. These valves are the shutdown service water (SX) system outlet valves to the hydrogen control system mixing compressor cubicle coolers 1VR09S and 1VR12S. A previous modification removed the hydrogen compressor cubicle doors. As a result, the cubicle coolers are no longer needed and valves 1SX095A and B are not required to operate. These valves do not have a containment isolation function and the non-1E classification will not degrade a 1E bus because double breaker protection is provided.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0002 PM VC-42

DELETE AUTO ACTUATION OF VC SYSTEM CHLORINE MODE

This plant modification deletes the automatic actuation of the chlorine mode of the control room ventilation (VC) system. The current Technical Specifications do not require automatic actuation when chlorine is stored at least 100 meters away from air intakes in containers having a capacity of 150 pounds or less. Presently, chlorine containers at Clinton Power Station are 150 pounds and are stored more than 100 meters away from the air intakes of the control room ventilation system. The chlorine monitors will still provide a main control room alarm for high chlorine concentration so that the chlorine mode can be manually actuated.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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TITLE

87-2660 PM VL-05

INSTALL DOP INJECTION MANIFOLD

This modification installs a permanent dioctyl phthalate (DOP) injection manifold for the testing of laboratory exhaust filter units OVL11SA and B. This modification has no impact on the operability of the filter units. The manifold is located in negative pressure ductwork which has no leak testing criteria. Any small leakages caused by the installation of this modification will not prevent the laboratory exhaust filter units from performing their design function.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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1988

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LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0039 TEMP MOD 88-011 INSTALL STRAINERS IN SUCTION LINES OF RADWASTE PUMPS

This temporary modification installs strainers in the suction lines of waste sample pump A and excess water pump A of the radwaste equipment drain (WE) system. The pumps are used to transfer water to the service water system and the cycled condensate system, or off-standard water back to the WE surge/collector tanks. The installation of a strainer on the suction of each pump will prevent resin migration into other systems. These pumps and their suction lines do not impact or interface with a safety-related system. The modification will not create the potential for an unexpected or uncontrolled release of radioactivity to the environment.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0061 TEMP MOD 88-014 REMOVE SMOKE DETECTORS FROM VC SYSTEM INTAKE AREA

This temporary modification removes two ionization smoke detectors from the control room ventilation (VC) system intake area in the control building. Water from the outdoors wets these two detectors causing all 30 detectors in the zone to become inoperable. These two detectors are not required because they are installed in an area that contains no combustible material and no ventilation equipment. Also, these detectors do not provide any automatic actuation. Their removal improves operability of the remaining detectors in this zone.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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NUMBER EVALUATED

TITLE

K5-0073 TEMP MOD 88-017 CROSS TIE WM SYSTEM LIME FEEDERS TO SOFTENERS

This temporary modification connects the makeup demineralizer (WM) system lime feeder A to lime softener B and lime feeder B to lime softener A. This allows lime softener B to remain in service while lime feeder B is repaired. Cross tying the WM lime feeders and softeners will increase system availability. This will ensure a continued supply of high quality water for continued plant operation. This modification does not impact the safety design basis of the plant because the WM system performs no safety related function. Also, failure of the WM system will not impact operation of any safety related system. This equipment is located in the makeup water pump house which is southwest of the plant service building.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0066 TEMP MOD 88-019 INSTALL BLOWERS IN THE TURBINE BUILDING MAIN STEAM TUNNEL

Higher than normal temperatures have been occurring in the turbine building main steam tunnel due to the cooling coils in the area being clogged, to minor steam leaks, and to some temperature sensors being located in a stagnant area. This temporary modification installs temporary portable blowers in the steam tunnel to eliminate air stratification. The blowers are powered from a non-IE source and their failure will not impact the safety function of any safety related equipment.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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TITLE

88-0081 TEMP MOD 88-020 ALTERNATE COOLING WATER SUPPLY FOR CONTROL ROD DRIVES (CRD)

During planned outage PO-2, the CRD pumps will be out of service. This temporary modification provides an alternate cooling water supply to the CRDs by installing four hoses from the cycled condensate (CY) system hose stations to the CRD supply line high point vent/low point drains. The portions of the CY and CRD systems which are affected by this modification are not safety related and the safety function of the CRD system is not affected. Failure of all four hoses will result in a loss of cooling water to the CRDs. However, the CRDs can function without cooling water, although seal life will be shortened. Also, multiple check valves will prevent reactor coolant inventory from leaking past the seals if seal cooling water is lost and the seals degrade. The system will be restored to its original configuration at the end of PO-2.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0091 TEMP MOD 88-021 INSTALL SPARE MOTOR OPERATOR ON RH SYSTEM VALVES

During planned outage PO-2, the motor operators on residual heat removal (RH) system valves 1E12-F024A and B will be removed and identical operators installed without electrical terminations. This will allow the valves to be manually operated. These valves isolate the RH test return lines to the suppression pool. This modification will allow the RH system to remain operable as required by the Technical Specifications. This modification will be performed with the Plant in Mode 4, and thus the automatic containment isolation function of these valves is not required.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

CLINTON POWER STATION
10CFR50.59 REPORT
FOR
PROCEDURES, TESTS AND
AND EXPERIMENTS, AND OTHER DOCUMENTS
FROM FSAR AMENDMENT 38 TO
RECEIPT OF THE OPERATING LICENSE AND
FROM JANUARY THROUGH MARCH

1988

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AND FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0064 2810.07 R.0

EMERGENCY BREATHING AIR FILTER LEAK TEST

This new test procedure controls the testing of air inleakage of the emergency breathing air filter unit, GRA02S. This procedure requires the installation of blanks at the inlet and outlet of the filter enclosure and subjects the filter unit to vacuum conditions which are well within its design capability. The breathing air system is not safety related and the filter unit is not used during normal plant operation.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0059 2851.01 R.0

RUSKIN FIRE DAMPER DROP TEST

This test procedure performs a drop test on Ruskin fire dampers in the following ventilation systems: auxiliary building, control room, screenhouse and makeup water pump house, machine shop, drywell purge, turbine building, radwaste building, and switchgear heat removal. This test requires that an electrothermolink be installed in place of a permanent fusible link, and that a transparent plastic viewing window be installed in place of a permanent inspection door. The testing will be performed so as to maintain system operability as required by the Technical Specifications. Installation of the electrothermolinks and the plastic viewing windows will not impair fire protection. After completion of the test, the systems will be restored to their original configuration.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0020 CR 1-88-01-035 FLUSH LINE FOR SERVICE WATER PUMP 1WS01PA AND C UPPER BEARINGS

This condition report identifies two discrepancies, one for the service water (WS) pumps 1WS01PA and C flush lines connections to the WS pump upper bearings and the other for a radially drilled hole in the WS pump shafts. The original design required a 1" flushing line to be reduced to 3/4" and finally connected to a 1/2" line. The as-built configuration does not include the 3/4" reducer. The original design also required that a 1/4" radial hole be drilled in the WS pump shafts to provide a flushing water flow path for the pump bearings. Instead, a 1/8" hole was drilled and countersunk to 1/4". This condition report justifies the acceptability of the flushing line and radial hole in their present configuration. Increasing the size of the flush line from 3/4" to 1" does not adversely alter the flow to the pump upper bearings. The 1/8" radial hole allows adequate flushing water flow and does not impair the fatigue strength of the pump shaft.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0067 CR 1-88-02-033 REVISE DRAWING TO REFLECT AS-BUILT CONFIGURATION OF VENT VALVE

The piping and instrumentation diagrams (FSAR Figure 9.2-2) presently show the shutdown service water system high point vent line 1SX127AA upstream of valve 1SX090A. This condition report corrects the diagram to show the vent as being downstream of 1SX090A, which is the as-built configuration. The vent is properly installed at the system highpoint which is the design requirement.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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AND FROM JANUARY THROUGH MARCH 1 '88

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0030 EMER PLAN R7

GENERAL REVISION OF THE CPS EMERGENCY PLAN

The Clinton Power Station (CPS) Emergency Plan was developed to support the Emergency Preparedness Program. This program has been established to prepare for and to respond to an emergency situation at CPS. This revision makes extensive changes to the Plan such as, revising position titles and responsibilities, clarifying definitions, relocating and redefining certain facilities and equipment, and incorporating clerical changes. These changes comply with the requirements of 10CFR50.47 and 10CFR50 Appendix E.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0031 EPIP AP-06 R2

REVIEW OF EMERGENCY PREPAREDNESS PROGRAM

This Emergency Plan Implementing Procedure (EPIP) ensures that the Emergency Plan and its associated implementing procedures are reviewed and that revisions are made to correct inadequacies identified during the review. This revision deletes references to temporary change notices, clarifies what information is required in the Emergency Preparedness Program Annual Review Report, adds the Nuclear Review and Audit Group (NRAG) as its recipient, and updates personnel titles in accordance with recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the Final Safety Analysis Report (FSAR). The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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88-0079 EPIP AP-07 R3 ALERT AND NOTIFICATION SYSTEM

This Emergency Plan Implementing Procedure (EPIP) provides guidance for documenting the performance of the alert and notification system (ANS) with respect to testing and maintenance. This revision corrects references to Attachment 4, "Siren Maintenance/Repairs Report", corrects the location of siren #15 due to relocation, and clarifies the requirements for providing siren maintenance information to the Director - Emergency Response. Also, guidelines for determining siren operability have been modified to conform with system design, documents comprising a monthly performance report have been clarified, and personnel titles have been corrected in accordance with recent management reorganization. Relocation of siren #15 does not impact the design of the ANS because its new location meets the requirements for notification of the public. The position title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0077 EPIP AP-09 R2 EMERGENCY COMMUNICATIONS OPERABILITY CHECKS

This Emergency Plan Implementing Procedure (EPIP) ensures that CPS emergency communications equipment will be checked periodically for proper operation and availability. This revision adds communications tests for the emergency response facilities, clarifies the retention of records, modifies definitions, clarifies satisfactory instrument performance, clarifies the purpose section, adds call-back verification on the Emergency Notification System (ENS), and adds roll call on the Nuclear Accident Reporting System (NARS). This revision also corrects personnel titles in accordance with recent management reorganization. The position title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG NUMBER	DOCUMENT EVALUATED	TITLE
87-2669	EP1P EC-04 R2	ALEFT

This Emergency Plan Implementing Procedure describes the actions to be taken in response to an "Alert" emergency classification. This revision changes the phrase "Emergency Action Level Classification" to "Emergency Classification", revises the text format, provides direction regarding preparation of emergency close-out reports, and corrects personnel titles in accordance with recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

87-2668	EP1P EC-05 R2	SITE AREA EMERGENCY
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This Emergency Plan Implementing Procedure describes the actions to be taken in response to the "Site Area Emergency" emergency classification. This revision changes the phrase "Emergency Action Level Classification" to "Emergency Classification", provides direction regarding the preparation of emergency close-out reports, revises the text format, and corrects personnel titles in accordance with recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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87-2667	EPIP EC-06 R2	GENERAL EMERGENCY
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This Emergency Plan Implementing Procedure describes the actions to be taken in response to a "General Emergency" emergency classification. This revision revises the definition of General Emergency, and changes the phrase "Emergency Action Level Classification" to "Emergency Classification". In addition, this revision provides direction regarding preparation of emergency closeout reports, revises text format, and corrects personnel titles in accordance with recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0098	EPIP FT-01 R3	TECHNICAL SUPPORT CENTER OPERATIONS
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This Emergency Plan Implementing Procedure describes the performance of activities in the Technical Support Center (TSC). This revision modified text to improve compatibility with related procedures, adds a requirement to connect the health physics network (HPN) telephone upon TSC activation, revises the definition of contaminated, and corrects personnel titles in accordance with recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR.50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0097 EPIP FE-03 R2

EMERGENCY OPERATIONS FACILITY OPERATIONS

This Emergency Plan Implementing Procedure (EPIP) describes the performance of activities in the Emergency Operations Facility (EOF). This revision modifies text to improve compatibility with related procedures, adds a requirement to connect the health physics network (HPN) telephone upon EOF activation, revises the definition of contaminated, and corrects personnel titles in accordance with recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0096 EPIP FE-04 R2

BACKUP EMERGENCY OPERATIONS FACILITY OPERATIONS

This Emergency Plan Implementing Procedure (EPIP) provides guidance on operations at the Backup Emergency Operations Facility (BEOF) once it is determined that the EOF is no longer habitable. This revision clarifies transfer of command authority, provides additional guidance on the information required during notifications, revises the list of members required to relocate to reflect only key personnel, changes the definition of activation to clarify when the facility is ready, clarifies access to BEOF, adds a checklist for activation, corrects the street address of the BEOF, and changes position titles in accordance with recent management reorganization. The address change and personnel title changes require a revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0036 EPIP FE-06 R2 EMERGENCY COMMUNICATIONS EQUIPMENT

This Emergency Plan Implementing Procedure (EPIP) provides descriptions of the communications systems available and the proper techniques to be employed when conducting communications in an emergency. This revision deletes unnecessary definitions, simplifies information regarding emergency communications systems, updates the dial code for the Nuclear Accident Reporting System when a "General Emergency" is declared, revises Attachment 2, "Emergency Radio System(s)", to improve usefulness, and updates personnel titles in accordance with the recent management reorganization. The changes to the communications systems and to the personnel titles require a revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0040 EPIP RA-01 R3 MANUAL RADIOLOGICAL DOSE ASSESSMENT

This Emergency Plan Implementing Procedure (EPIP) is a manual radiological dose assessment procedure used as a backup to the computerized dose assessment method. This revision adds time correction factors and count rate correction factors for the accident effluent monitors, revises the dose conversion factors, adds channel addresses for the effluent monitors, allows doses to be calculated for unfiltered ventilation releases, and corrects position titles in accordance with the recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0095 EPIP RA-08 R2 RADIOLOGICAL INJURIES

This Emergency Plan Implementing Procedure provides guidance when handling and transporting personnel who are or may be radiologically contaminated because of an injury within a Radiological Controlled Area. This revision revises the procedure format; provides information for generating accident reports; revises the definitions of contaminated, serious injury, and minor injury; replaces the term patient with victim; revises references; revises the scope of first aid for minor injuries; and corrects personnel titles in accordance with the recent management reorganization. The personnel title changes require revision to the Emergency Plan, which is part of the FSAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0207 FPER AMEND 2 FIRE PROTECTION EVALUATION REPORT AMENDMENT

The Fire Protection Evaluation Report (FPER) is being amended to incorporate "as-built" information concerning fire protection features of the plant. This amendment includes revisions to the combustible loadings, the minimum wall thicknesses, the fire barrier ratings, the number of floor drains, the floor areas (square feet), the detection/suppression systems, and the fire protection feature drawings. These changes comply with the requirements of 10CFR50 Appendix R and do not increase the fire loadings above the ratings of fire walls for any fire area, physically alter any fire barrier, affect the design drainage capability, or adversely change the detection/suppression systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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TITLE

88-0088 FPER FIG FP-178 DERATING OF RADWASTE BUILDING FIRE RATED WALL

Figure FP-176 in the CPS Fire Protection Evaluation Report (FPER) and Safe Shutdown Analysis (SSA) shows a 1.9 hour fire rated wall surrounding a stairwell and elevator at approximately column 124.9, row H.9, elevation 720'-6" in the radwaste building. Penetration 1SR2126 is located in this wall at elevation 723'-1 1/2" and does not have a fire rated damper. The portion of Figure FP-178 will be revised to show the wall surrounding the penetration as non-fire rated. Derating the fire rated wall is acceptable because the combustible load is estimated to be within acceptable limits, the wall is located in a non-safety related building with no safe shutdown significance, and there is another enclosed stairwell with fire rated walls located in this building.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0263 FSAR 1.8

VENTILATION FILTER EXCEPTIONS TO REGULATORY GUIDE 1.140

This FSAR change identifies paragraphs of Regulatory Guide 1.140, Rev. 0, with which Clinton Power Station does not comply. Paragraphs C.3.e and C.4.b of the regulatory guide provide guidance on the maintenance and removal space required for ventilation filter components. Although the specific requirements of the regulatory guide are not met by the radwaste building ventilation system and the drywell purge system, adequate maintenance space and access are provided as a result of the specific physical configuration of the filter units in these systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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87-2659	FSAR 13.2	TRAINING
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Section 13.2 of the FSAR describes and outlines various training programs at Clinton Power Station which are structured to comply with the applicable regulatory requirements. The changes made in Section 13.2 of the FSAR involve the Licensed Operator Requalification Program and are a result of changes to the program because of revisions made to 10CFR55 and Regulatory Guide 1.8, Rev. 2. This FSAR change will reflect that the CPS training program is in compliance with newly revised regulatory guidelines.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0227	FSAR 14.2.12.2	REACTOR RECIRCULATION PUMP COASTDOWN REQUIREMENT
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The acceptance criteria for the reactor recirculation pump coastdown test in FSAR section 14.2.12.2.27.4B incorrectly requires that the flow coastdown transient be equal to or faster than the limiting curves. Since the limiting curves defined by the Transient Safety Analysis Design Report are maximum and minimum curves, the coastdown transient result must be between the two. This FSAR change corrects the pump coastdown criteria and ensures that the results of the pump coastdown test are consistent with the design analysis values.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
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TITLE

88-0248 FSAR 14.2.12.2 REVISE MAIN STEAM LINE FLOW VENTURI DIFFERENTIAL PRESSURE

This FSAR change revises the startup test Level 2 acceptance criteria for the main steam line flow venturi differential pressure. The value specified in the FSAR is incorrect and is being revised from 84.60 pounds per square inch to 79.71 pounds per square inch. The revised value is consistent with original design specification of the venturi. No change is being made to the function and physical configuration of the venturi.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0235 FSAR 14.2.12.2 SOURCE RANGE MONITOR COUNT RATE CRITERIA

This FSAR change revises the source range monitor (SRM) count rate criteria in the following startup tests: the SRM performance test and the control rod sequence test Level 1 acceptance criteria; and the fuel loading and the full core shutdown margin test prerequisites. The requirement for a minimum SRM count rate of 0.7 counts/second is being revised to state that there must be neutron signal count-to-noise count ratio of at least 20:1 on the required operable SRMs with a minimum count rate of 0.7 counts/second; otherwise, there must be a neutron signal count-to-noise count ratio of at least 2:1 with a minimum count rate of 3.0 counts/second. This change brings the FSAR into conformance with the more stringent requirements of the Technical Specifications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0183 FSAR 2.1.1.3 BOUNDARIES FOR ESTABLISHING EFFLUENT RELEASE LIMITS

This FSAR change revises the name of the restricted area to be the owner controlled area (OCA) and removes a statement requiring periodic surveillances of the OCA. Also, the description of the cyclone fence (that fence which establishes the boundary of the OCA) was revised from "preventing access" to "restricting access" to the OCA. These changes do not affect plant radiation protection, the control of access to the Protected Area, or security systems. There are no regulatory requirements for maintaining periodic surveillances or for preventing access to the OCA.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0024 FSAR 2.3.3.2 DELETE CHECK OF RECORDED AIR TEMPERATURE

FSAR section 2.3.3.2 provides a description of the maintenance and calibration of meteorological instrumentation. The FSAR requirement to check recorded air temperatures against values obtained on the tower with American Society for Testing and Materials (ASTM) precision thermometers is being deleted. This requirement is redundant since temperature sensors are calibrated semi-annually using an ice bath.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0084	FSAR 2.5.4.13	SETTLEMENT MONITORING PROGRAM

FSAR section 2.5.4.13 and Figure 2.5-438 describe the plant settlement monitoring program and graph the settlement data, respectively. The plant settlement monitoring program was instituted to track plant settlement and monitor excessive or differential settlement. Review of settlement data shows that plant settlement has stabilized well within the predicted design range. FSAR Figure 2.5-438 will be revised to update plant settlement graphs, and FSAR section 2.5.4.13 will be revised to discontinue the settlement monitoring program.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0224	FSAR 3.5.1.7	INTERNALLY GENERATED MISSILES FROM PRESSURIZED COMPONENTS
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This FSAR change revises the description of the methodology used to analyze the effects of internally generated missiles from pressurized components. Single nut, bolt, nut and bolt, and nut and stud combinations are being excluded from consideration as potential missiles since the stress levels to which these bolts and studs are torqued result in a strain energy which is too low to permit the nut, bolt or stud to become a missile. Also, the 5 foot radius criteria for thermowell impact analysis was eliminated so that all potential impacts are analyzed, including those beyond a 5 foot radius. FSAR Table 3.5-9 is being revised to show the results of the revised thermowell impact criteria. The revised table shows that no potential thermowell missiles could hit and disable components which are essential for safe shutdown of the plant.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

88-0016 FSAR 5.4.5.4

MAIN STEAM ISOLATION VALVE TESTING

FSAR section 5.4.5.4 describes the inspection and testing of the Main Steam Isolation Valves. The level of detail provided is not required by Regulatory Guide 1.70, Rev. 3, the Standard Review Plan (NUREG 0800), and is not discussed in the Safety Evaluation Report (NUREG 0853) or its supplements. This FSAR change deletes detail which is not required to be in the FSAR.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0289 FSAR 6.4

REVISE CONTROL ROOM VENTILATION SYSTEM DESCRIPTION

This FSAR change revises the description of the control room ventilation (VC) system to include a description of the locker room exhaust fan, the locker room exhaust fan trip logic, and the maximum outside air purge damper trip logic. This FSAR change also revises the humidity control zones of the VC system and specifies that a positive pressure is not maintained in the control room when the VC system is operating in the maximum outside air purge mode. These revisions do not affect the ability of the VC system to maintain control room habitability during its normal and accident modes of operation.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0286	FSAR 6.4, 9.4	DESIGN AND TESTING OF THE CONTROL ROOM VENTILATION SYSTEM

FSAR sections 6.4 and 9.4 describe the design and testing of the control room ventilation (VC) system. These sections are being revised to indicate that the control room is divided into seven independent environmental control zones instead of six; that the cooling capacity of the VC system was verified by a temperature survey instead of a heat balance; and that the positive pressure maintained by the VC system is 1/8" water gauge instead of 1/4" water gauge. These changes do not affect the ability of the VC system to maintain the habitability of the control room.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0229	FSAR 6.5.1	SGTS, VR SYSTEM, AND DRYWELL PENETRATION DESCRIPTIONS
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This FSAR change revises the descriptions of the standby gas treatment system (SGTS), the containment building ventilation (VR) system, and drywell penetrations. The calculated capacity of the SGTS equipment train standby cooling air fan in FSAR section 6.5.1.1.1 is being revised from 12,300 BTU/hr to 12,700 BTU/hr. This change is the result of a revised calculation which demonstrates an increased margin in the fan's capacity for heat removal. Also, the integrated dose for the once-in-lifetime post loss of coolant accident scenario, with respect to the control room ventilation system makeup air filter packages in FSAR section 6.5.1.6, is being revised from 2E+08 rads to 1E+05 rads to agree with the original design specification. Finally, the description of drywell penetrations LMD-154 and LMD-194 in FSAR Table 3.8-5 is being revised to denote that the penetrations are being used for the containment monitoring system. This will correct a discrepancy between the table and the piping and instrumentation diagram (FSAR Figure 7.6-23) which shows the correct configuration.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0237 FSAR 6.7

TIMER AND PRESSURE SENSOR ADDITION TO THE MSIV-LCS

This FSAR change revises section 6.7.2 and Table 6.7-1 to reflect the addition of a timer and a pressure sensor to the downstream portion of the main steam isolation valve leakage control system (MSIV-LCS). The timer adds a 6 minute initiation delay and, together with the pressure sensor, provides a permissive signal which allows the downstream MSIV-LCS to be transferred from the depressurization mode to the bleed off mode of operation. This permissive signal will ensure that the main steam lines are depressurized prior to the system being aligned to the MSIV-LCS blowers so that blower damage is avoided. The new instrumentation improves the overall ability of the MSIV-LCS to process leakage.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0232 FSAR 6.7.3.5

MAIN STEAM ISOLATION VALVE LEAKAGE CONTROL SYSTEM

Flow limiters in the main steam isolation valve leakage control system bleed lines are used to automatically isolate the bleed lines from their associated steam line at a given flow rate. This FSAR change updates the range of typical flow rate setpoint values by referring to the actual setpoints. The actual setpoints are specified in the Technical Specifications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0181 FSAR 7.2

REACTOR PROTECTION SYSTEM

This FSAR change revises the description of the reactor protection (RP) system reset switch and the setpoint of the turbine stop valve and turbine control valve fast closure. The description of the reset switch is being revised to state that manual reset is prohibited for 10 seconds after a single trip logic is tripped. This will ensure that the RP system fully responds to the trip signal prior to it being reset. The setpoint of the turbine stop valve and the turbine control valve fast closure is being revised to 40% of rated reactor power. This setpoint is consistent with the original design of 30% turbine shell pressure.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0247 FSAR 7.3.1.1.2 REVISE METHOD OF MAIN STEAM LINE AREA LEAK DETECTION

This FSAR change revises section 7.3.1.1.2.4.1.3.1 to correct the description of the method used to detect a main steam line leak in the main steam tunnel. The revised description states that the temperature difference of the water entering and leaving the air cooling units in the main steam tunnel will be monitored instead of the air temperature difference. These methods of main steam line leak detection are equivalent because an increase in heat gain to the main steam tunnel can be detected by a change in the temperature difference of the cooling water across the cooler. Monitoring the temperature difference of water entering and leaving the air cooling units is the common method of steam line leak detection used in areas outside the containment.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0266 FSAR 7.4.1.4.4 INSTRUMENTATION ADDITIONS TO THE REMOTE SHUTDOWN PANEL

This FSAR change revises the description of the remote shutdown panel to reflect the addition of a suppression pool temperature indicator and a control switch to the panel. The control switch operates the shutdown service water system bypass valve to residual heat removal system heat exchanger 1A. These changes to the remote shutdown panel enhance the operator's ability to bring the plant to a safe shutdown when the main control room is uninhabitable. The added components have been designed to meet the original seismic, environmental, and electrical specifications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0290 FSAR 7.7, 9.4 DELETE CONTAINMENT PRESSURE INTERLOCK IN VR/VQ SYSTEM

This FSAR change revises sections 7.7.1.16.1 and 9.4.7.2.2 to delete reference to a containment pressure interlock on the drywell purge (VQ) system/containment ventilation (VR) system isolation valves. The FSAR stated that these valves could not be opened unless the containment pressure was less than 3 psig. This interlock is not necessary as these valves are interlocked with high drywell pressure and loss of coolant accident signals, both of which will prevent the opening of the VQ/VR system isolation valves prior to the containment reaching 3 psig.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0265 FSAR 7.7.1.9

REVISE DESCRIPTION OF RADIATION MONITORING SYSTEM

This FSAR change makes minor revisions to the description of the radiation monitoring system monitors and the central control terminals to clarify the operation of the "as-built" configuration. No changes were made to the design or functional requirements of the system. Also, the functions of interfacing systems are not affected by these changes.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0257 FSAR 8.3.1.4.4

REVISION TO INSTRUMENT CABLE ROUTING CRITERIA

This FSAR change allows for deviations from the routing criteria for instrument and control cables. Presently, instrument cables that share raceways with control cables cannot be run parallel to the control cables unless the raceway is equipped with internal barriers which provide added electromagnetic shielding. Exceptions to this requirement would be allowed in individual cases when justified by supporting analysis which meets the criteria of IEEE Standard 384. This change does not adversely impact plant safety because each exception must be supported by an individual design analysis to confirm the technical adequacy of the cable routing.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0245 FSAR 9.1

DELETE FUEL CHANNEL GAUGING FIXTURE

This FSAR change deletes section 9.1.4.2.3.6 and Figure 9.1-10 which describes the fuel channel gauging fixture. The fixture was designed to be used in detecting unacceptable warpage of fuel channels. The fixture is being eliminated because industry experience with fuel channels similar to the fuel channels used at Clinton Power Station has shown that warpage is unlikely and that any warpage can be detected by a friction test. This friction test has been incorporated into procedure 2212.01, "Fuel Channel Management".

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0278 FSAR 9.1.4.2

REVISED USE OF REACTOR VESSEL HEAD STRONGBACK

This FSAR change revises the description of the reactor head strongback to allow its use when moving the drywell head to and from its storage location. The combined weight of the drywell head and the strongback is within the rated capacity of the containment polar crane. Also, the weight of the drywell head is less than the rated capacity of the strongback.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0288 FSAR 9.4

REVISE VR SYSTEM AND VQ SYSTEM DESCRIPTIONS

This FSAR change clarifies the operation of the containment building ventilation (VR) system and the drywell purge (VQ) system following a loss of power or instrument air. The FSAR currently states that a single switch is provided to reopen the VR/VQ system isolation valves after a loss of power or instrument air. Since certain modes of VR/VQ system operation require that several combinations of switches be manipulated to reopen the isolation valves, the FSAR is being revised to state that operator action is required to reopen the valves. This FSAR change also revises the description of the VQ system charcoal bed deluge system. The FSAR currently states that deluge of the charcoal beds is automatic when high temperature is sensed. This description is being revised to state that the deluge is not automatic, and that high and high-high alarms are annunciated in the control room and subsequent operator action is required to actuate the deluge. Automatic deluge of the charcoal filter units is not required to satisfy regulatory requirements. Also, temperature alarms are conservatively set to provide ample time for operator response.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0267 FSAR 9.4.5.2.1 REVISION OF SWITCHGEAR ROOM TEMPERATURES

This FSAR change revises the maximum allowable temperatures in the switchgear rooms from 84 degrees F to 95 degrees F during normal operation and from 99 degrees F to 104 degrees F during abnormal operation. The new allowable temperatures will not impact equipment operation because they are still less than the temperature to which the equipment in the switchgear rooms is environmentally qualified.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0225	FSAR 9.4.6.2.1G	CONTAINMENT BUILDING VENTILATION SYSTEM DESCRIPTION

This FSAR change corrects an inaccurate statement concerning radioactive effluents released through the containment building ventilation system from the containment building refueling floor. The statement currently reads that "...no contaminated air is exhausted to the outside..." because the time to isolate the exhaust after detection of high radiation is 6 seconds and air travel time through the ducts is 9.9 seconds. Considering the sensitivity of the radiation monitors, this FSAR statement has been corrected to be more accurate. The FSAR has been changed to read "...release of contaminated air to the outside atmosphere is either precluded or is insignificant." The insignificant release does not affect any accident evaluation or the consequences of an accident.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0044	FSAR APP D	REPORT ON OUTAGES OF EMERGENCY CORE COOLING SYSTEMS
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This change revises the response to Three Mile Island (TMI) Action Plan II.K.3.17 to clarify Illinois Power Company's position for reporting emergency core cooling system (ECCS) outages. The revised response states that ECCS outages will be reported via participation in the Nuclear Plant Reliability Data System (NPRDS). Participation in the NPRDS meets the intent of TMI Action Plan Item II.K.3.17, and is acceptable to the NRC.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE AND FROM JANUARY THROUGH MARCH 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0268 FSAR FIG 5.1-3 REMOVE POWER FROM HIGH/LOW PRESSURE INTERFACE VALVES

This FSAR change adds a note to Figure 5.1-3 to indicate that valve 1B21-F001 will have its electrical power disconnected during normal operation to prevent it from spuriously opening. This valve is a normally closed reactor head vent valve. If the valve were to spuriously open while the reactor is operating at high pressure, the lines down stream of the valve would be subjected to higher pressures than for which they were designed. Also, valves 1F12-F052A & B will have their power disconnected during power operation to prevent them from spuriously opening. These valves are the steam supply line isolation valves to the residual heat removal system heat exchangers. The above changes are the result of the spurious operations analysis performed as part of the CPS Fire Protection Shutdown Analysis. Administrative control of power to these valves has been implemented in procedures 3006.01 and 3312.01 to ensure that power is provided to the valves when they may be required to operate.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0231 FSAR FIG 5.4-13 RESIDUAL HEAT REMOVAL SYSTEM LOOP B MINIMUM FLOW LINE

During system vibration testing, it was determined that the residual heat removal (RH) system loop B minimum flow line had excessive vibration caused by a single-hole orifice installed in that line. The single-hole orifice was replaced with a multi-hole orifice and subsequent testing showed that the vibration had been reduced to acceptable levels. The new multi-hole orifice meets all design requirements of the original orifice including the minimum flow requirement. This FSAR change revises the piping and instrumentation diagram to show the multi-hole orifice.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0175 FSAR FIG 6.7-3 MAIN STEAM ISOLATION VALVE LEAKAGE CONTROL SYSTEM

This FSAR change deletes Figure 6.7-3 and references to the figure in the text. This figure is the functional control diagram for the main steam isolation valve (MSIV) leakage control system (IS). The information contained in the figure is also contained in the CPS electrical schematic diagrams. The electrical schematic diagrams for the IS system are identified in FSAR Table 1.7-1. In general, functional control diagrams are not included as FSAR figures.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0283 FSAR FIG 8.3-2A REVISIONS TO HPCS POWER SUPPLY INSTRUMENTATION

This FSAR change revises Figure 8.3-2a to show the potential transformer fuse in the high pressure core spray system (HPCS) power supply being 0.5 amps instead of 1 amp. This corrects the fuse size to be the size recommended by the manufacturer and ensures that proper protection is provided for the transformers. This change also revises the phase that the synchroscope is connected to from A to C. The change to the synchroscope connection has no impact on its operation since it can work properly no matter which phase it is connected to.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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TITLE

87-2663 FSAR FIG 8.3-3 RESIDUAL HEAT REMOVAL SYSTEM PUMP START TIMES

The start timing sequence for residual heat removal (RH) system pumps 1A and 1B is 5 seconds after closure of the diesel generator circuit breaker. RH pump 1C starts 0 seconds after breaker closure. FSAR Figure 8.3-3, sheet 1, shows a 5 second start time delay for all three RH pumps. The figure will be corrected to show the proper starting delays as indicated above. This change will not adversely impact plant safety because the start time delay for RH pump 1C was conservatively changed from 5 seconds to 0 seconds. Also, the generator load curve reflects the 0 second time delay for pump 1C.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0221 FSAR FIG 9.1-13 FUEL CHANNEL HANDLING BOOM

This FSAR change revises Figure 9.1-13 which shows the fuel channel handling boom. The mounting detail of the boom was changed from a floor socket mount to a floor mounted base plate. The new base plate design provides support equivalent to the original socket design. Previous functional and accident analysis are not impacted by this change.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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88-0251 FSAR FIG 9.2-2 ADDITION OF ORIFICES TO SHUTDOWN SERVICE WATER SYSTEM

This FSAR change revises Figure 9.2-2, sheets 1 and 2, to show the addition of flow restricting orifices in the shutdown service water system discharge pipes of the residual heat removal system A and B heat exchangers. These orifices were added to reduce high flow conditions exhibited during testing in order to achieve the original design flow. The design of these orifices meets the piping design criteria for this system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0249 FSAR FIG 9.2-2 ADDITION OF ORIFICES TO SHUTDOWN SERVICE WATER SYSTEM

This FSAR change revises Figure 9.2-2, sheets 1 and 2, to show the addition of flow restricting orifices in the shutdown service water system discharge pipes of the fuel pool cooling and cleanup heat exchangers A and B. These orifices were added to reduce high flow conditions exhibited during testing in order to achieve the original design flow. The design of the orifices meets the piping design criteria for this system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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TITLE

88-0269 FSAR FIG 9.2-4 ADD INTERTIE FROM WM SYSTEM TO TF SYSTEM

This FSAR change revises the piping and instrumentation diagram for the make-up water demineralizer (WM) system to show an intertie with the floor drain (TF) treatment system. This intertie allows regenerative waste from the makeup demineralizers to be routed to holding tanks where it can be neutralized prior to its release to the treatment pond. The holding tanks are located north of the radwaste building. System failure will not impact any safety related equipment.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0242 FSAR FIG 9.3-2 REPLACE AIR OPERATED VALVES IN THE LIQUID RADWASTE SYSTEM

This FSAR change revises Figures 9.3-2 and 11.2-2 to show that the air operated valves for the main radwaste filter drain valves OWELL8A, B, and C have been replaced with quick exhaust shuttle valves. The function of the air operated valves is to open the main drain valves for backwashing. The new valves provide faster opening times which are needed to achieve the "air bump" effect for an effective backwash. Also, the new valves meet the required seismic, quality group, and Quality Assurance criteria. Design provisions for controlling radioactive releases are not affected. This change does not increase the amount or alter the flow of radioactive fluids.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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TITLE

88-0271 FSAR FIG 9.3-3 REVISE MODEL OF BREATHING AIR SYSTEM MANIFOLDS

This FSAR change reflects a change in the model of the breathing air system manifolds. The original manifolds were replaced with a model compatible with the radiation protection respiratory equipment. This change corrects the problem of respirators being incompatible with the installed manifolds. The new breathing air manifolds perform the same function as the old manifolds.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0241 FSAR FIG 9.4-11 RWCU ROOM SUPPLY AIR BACKDRAFT DAMPER EQUIPMENT NUMBER

FSAR Figure 9.4-11 incorrectly identifies the equipment number of the reactor water cleanup (RWCU) system valve room B backdraft damper as being 1VR43Y. This change revises the damper number to 1VR145Y, and thus provides an accurate description of the containment building ventilation system. The original design intent of the damper is not changed.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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88-0291 FSAR Q&R 410.7 FIRE DOOR COMPLIANCE TO 10CFR50 APPENDIX R

This FSAR change clarifies an exception to 10CFR50 Appendix R criteria by making a distinction between the swing-type fire doors and the rolling steel fire doors that are used in the diesel generator bays. Unlike the swing-type doors, the rolling steel doors are not self-closing, and the rolling steel door to the Division III diesel generator room is not locked. These rolling steel doors must be kept closed in order for the fire suppression systems in the diesel generator bays to function properly. To ensure that the rolling steel doors are maintained closed, the doors to the Division I and II diesel generator bays are locked closed and inspected weekly, while the door to the Division III diesel generator bay is inspected daily. Also, in order to gain access to the Division III door, personnel must enter the Division I bay which is locked. Finally, a fire watch is posted whenever a rolling steel door is to be left open.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0005 FSAR TAB 11.5-4 SUMMARY OF RADIOLOGICAL ANALYSES OF LIQUID PROCESS SAMPLES

FSAR Table 11.5-4 provides a summary of radiological analyses performed on liquid process samples. This table is not referenced in any FSAR text and is being deleted from the FSAR. The information provided in this table is not required in the FSAR per the Standard Review Plan (NUREG-0800) Section 11.5 or Regulatory Guide 1.70, Rev. 3.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10C'R50.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
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88-0201 FSAR TAB 3.10-1 DELETION OF EQUIPMENT QUALIFICATION TABLES

FSAR Tables 3.10-1, 3.10-2, 3.11-1 and 3.11-2 list balance of plant (BOP) and nuclear steam supply system (NSSS) electrical equipment which requires seismic and environmental qualification. Tables 3.11-3 and 3.11-4 provide qualification parameters. Table 3.11-20 lists BOP and NSSS mechanical equipment which requires environmental qualification. These tables are being deleted from the FSAR, and all text references to the deleted tables are being changed to reference Nuclear Station Engineering Department (NSED) Maintenance Standard MS-02.00. According to Regulatory Guide 1.70 and sections 3.10 and 3.11 of NUREG 0800, these tables are not required to be part of the safety analysis report. Deletion of the tables does not release the currently qualified equipment from being maintained in its qualified condition. Any such change would be the result of a separately approved modification and would be individually evaluated and documented. The qualifications of currently qualified equipment will be maintained by Maintenance Standard MS-02.00.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0255 FSAR TAB 3.2-1 REVISE CLASSIFICATION OF DIESEL GENERATOR AIR START SYSTEM

This FSAR change revises the quality group classification for the vendor supplied skid mounted piping and valves of the air start systems for the Division I and II diesel generators from "C with no N Stamp" to "D" (these quality groups are defined in Regulatory Guide 1.26). This revision makes the classification of the skid mounted air start piping and valves consistent with the classification of air start piping mounted on the diesel engine. The failure analysis of the air start system shown in FSAR Table 9.5-10 is not affected by this change.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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88-0250 FSAR TAB 3.2-1 CHANGE IN CLASSIFICATION OF DG SYSTEM COMPONENTS

This FSAR change corrects three entries in Table 3.2-1 associated with the classification of the Division III diesel generator (DG) air start system components. The electrical classification of the air receiver tank is being changed to "N/A" because electrical classification is not applicable to tanks. The reference to note (J) in the "Comments" column of the air receiver tank is being deleted because note (J) applies to vendor skid piping and valves, and not to the air receiver tank. Finally, the "Quality Assurance Requirement" column for piping and piping components is being changed from "N/A" to "B". This change requires that piping and piping components conform to the Quality Assurance requirements of 10CFR50, Appendix B, which are consistent with their original design and installation.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0254 FSAR TAB 3.2-1 REVISE CLASSIFICATION OF RA SYSTEM COMPONENTS

This FSAR change revises the quality group classification for the control room emergency breathing air (RA) system from "C" to "C/D" (these quality groups are defined in Regulatory Guide 1.26). This change was made to allow the use of commercial grade air pressure regulators. The original "C" classification was not necessary to satisfy any regulatory requirement and was specified by Illinois Power Company to provide a higher degree of procurement control, installation, and testing of the RA system. The RA system is not safety related and is not required to mitigate the consequences of any analyzed accident.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG	DOCUMENT	TITLE
NUMBER	EVALUATED	
88-0228	FSAR TAB 3.7-10	SEISMIC LOADS OF REACTOR PRESSURE VESSEL AND INTERNALS

FSAR Table 3.7-10 provides a comparison of calculated seismic loads and allowable loads of the reactor pressure vessel and its internals. This FSAR change deletes Table 3.7-10 because the seismic and allowable load data was never listed. The required information is contained in Table 3.9-2 which is not being changed.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0190	FSAR TAB 3.9-2	REVISE STRESSES TO REFLECT AS-BUILT PIPING STRESS ANALYSES
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This FSAR table presents loading combinations, analytical methods, calculated stresses, and other design values for the most critical areas in the design of ASME Code Class 1, 2, and 3 components, including component supports and core support structures. Values in this table are being revised to incorporate the results of the as-built piping stress analyses. All of the revised values are still within the ASME Code allowables. Therefore, this FSAR change will not reduce the safety of the systems involved or any other system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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TITLE

88-0240 FSAR TAB 3.9-5 RECLASSIFICATION OF VC SYSTEM DRAIN VALVES

This FSAR change deletes 8 control room ventilation (VC) system drain valves from Table 3.9-5, "Balance of Plant Active Valves and Pumps," because their classification has been changed from active to passive. These drain valves open to allow fire protection water to drain from the VC charcoal filters after actuation of the fire protection deluge valves. Failure of the drain valves to open does not create a flooding concern because a plant operator is required to verify that the VC filters properly drain upon actuation of the fire protection deluge valves. If necessary, the operator can manually open drain valve.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0252 FSAR TAB 5.2-6 CORRECT UNITS OF MEASUREMENT FOR CONDUCTIVITY

This FSAR change corrects the units of measurement for conductivity in FSAR Table 5.2-6 from "mho/cm-25 degrees C" to "micromho/cm-25 degrees C". This change does not affect any plant equipment because all applicable equipment has been designed to achieve and measure in the "micro" range.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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88-0244 FSAR TAB 6.2-47 CLARIFICATION OF VR SYSTEM VALVE CLOSURE TIME REQUIREMENTS

This FSAR change adds a reference to note 29 of Table 6.2-47 for valves 1VR007A and B and 1VR006A and B. These valves are in the continuous containment purge (VR) system. The addition of this reference provides clarification that the source of the valve closure times and the requirements for the periodic surveillances are the ISI Program and the Technical Specifications, respectively.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0115 FSAR TAB 6.2-47 POST ACCIDENT SAMPLING SYSTEM VALVE SIZE CHANGE

This FSAR change revises the sizes of post accident sampling system valves in Table 6.2-47 and Figure 9.3-3 because of design changes that were made to allow the system to meet flow and leak rate test requirements. The new valves are seismically and environmentally qualified to the same requirements as the original valves.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0243	FSAR TAB 6.2-47	DELETE CLOSURE TIME OF VALVE 1C11-F083

This FSAR change revises the closure time for remote manual motor operated valve 1C11-F083 in the control rod drive system from "STD" to "N/A". This change makes the closure time for this valve consistent with the closure time of other remote manual motor operated valves. By design, valve 1C11-F083 is a remote manual operated valve, i.e., no automatic actuation. Closure time is not required to be specified because operator action is needed to close the valve.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0262	FSAR TAB 6.2-47	RELOCATE AND ADD VALVES TO INSTRUMENT AIR SYSTEM
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This FSAR change reflects the relocation of two relief valves (1IA128A & B) from the piping between the inboard and outboard containment isolation valves to a vendor supplied panel at the automatic depressurization system air supply bottles. The change also reflects the addition of a new relief valve (1IA128C) to the piping that supplies air from the instrument air system air amplifiers. The relocated valves will be removed from FSAR Table 6.2-47 because they are no longer containment isolation valves. Also, since they are not located within the containment boundary, they are being downgraded from ASME Section II Class 2 to ASME Section VIII. The relocation of the relief valves improves the containment boundary since there are now two fewer valves whose failure could affect containment integrity. Also, the valves remain classified "active" in the ISI program.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0264	FSAR TAB 6.5-3	VENTILATION SYSTEM COMPLIANCE TO REGULATORY GUIDE 1.52

This FSAR change revises Table 6.5-3 to expand the discussion of ventilation system compliance with the system, component, and environmental design criteria, the qualification testing, and the maintenance requirements of Regulatory Guide 1.52, Rev. 2. Although exceptions have been taken to specific requirements of the regulatory guide, the design and performance testing of the air filtration and absorption units which are part of engineered safety feature atmosphere cleanup systems (the control room ventilation system, the drywell purge system, and the standby gas treatment system) is adequate to ensure that they perform their design function.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0042	FSAR TAB 9.3-3	PROCESS SAMPLING TABULATION
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FSAR Table 9.3-3 is a tabulation of samples taken by the process sampling (PS) system. This change deletes alarm setpoints listed in the table because there is no requirement in the Standard Review Plan or Regulatory Guide 1.70 for these setpoints to be listed in the FSAR. FSAR 9.3-4 is also being revised to show that silica analyzers have been disconnected. The silica analyzers do not perform any safety function and do not interface with equipment important to safety. Silica monitoring will be performed via grab samples and laboratory analysis.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR59.59 REPORT FROM AMENDMENT 38 TO RECEIPT OF THE OPERATING LICENSE
AND FROM JANUARY THROUGH MARCH 1988

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NUMBER EVALUATED

TITLE

87-2658 NTD 2.13 R2 NTD LICENSED OPERATOR REQUALIFICATION PROGRAM

This Nuclear Training Department (NTD) procedure implements licensed operator requalification training as described in section 13.2.2.1 of the FSAR. This procedure revision incorporates changes made to 10CFR55 and includes a method for the development, control, and modification of study notes. These changes support the applicable regulatory requirements.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0223 O&R 640.13 FIELD TEST REQUIREMENTS FOR DOOR SEALS

This FSAR change revises the field test requirements for water tight door seals from pressurized air tests to chalk/grease fit-up tests. The door seal assemblies were leak tested by the manufacturer prior to installation at Clinton Power Station. After installation, a verification of proper surface contact between the door seal and the door frame will ensure that the door seal functions properly. For this reason, only the chalk/grease fit-up test is required. The chalk/grease test does not impact the operation or function of the door seals.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0206	SSA AMEND 2	SAFE SHUTDOWN ANALYSIS AMENDMENT

The Safe Shutdown Analysis (SSA) is being amended to incorporate "as-built" information concerning fire protection features of the plant. This amendment includes revisions to the Spurious Operations Analysis Table, the equipment lists, the combustible loadings, the minimum wall thicknesses, the fire barrier ratings, the number of floor drains, the fire protection feature drawings, and the fire protection deviation drawings. These changes comply with 10CFR50 Appendix R and do not increase fireloadings above the ratings of fire walls for any fire area, physically alter any fire barrier, affect the design drainage capability, decrease cable separation distances below Appendix R Section III.G or III.L criteria, or adversely change the detection/suppression systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0069	SSA APP A	SAFE SHUTDOWN ANALYSIS, APPENDIX A
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Appendix A of the Safe Shutdown Analysis (SSA) provides an analysis of CPS which shows that for any spurious electrically powered valve operation, safe shutdown capability is maintained. Appendix A of the SSA is being revised to include the analysis of bypass lines with motor operated valves installed around check valves. This concern was identified in NRC Inspection and Enforcement Notice 87-50. The results of this analysis concluded that this configuration does not affect the safety of the plant.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.