



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 3, 1998

Mr. Randall K. Edington
Vice President - Operations
Entergy Operations, Inc.
River Bend Station
P. O. Box 220
St. Francisville, LA 70775

SUBJECT: RIVER BEND STATION, UNIT 1 - RELIEF AUTHORIZATION FROM THE REQUIREMENTS OF ASME SECTION XI FOR THE SECOND TEN-YEAR INSERVICE INSPECTION INTERVAL, REQUESTS FOR RELIEF RR2-0001, RR2-0002 AND RR2-0003 (TAC NO. MA0620)

Dear Mr. Edington:

By letter dated January 7, 1998, Entergy Operations, Inc. (Entergy or EOI), requested relief from the requirements of ASME Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) for the second inservice inspection (ISI) interval at the River Bend Station (RBS), Unit 1 in accordance with Paragraph 50.55a(a)(3) of Title 10 of the Code of Federal Regulations, Part 50. Additional information was provided by Entergy in its letter dated July 8, 1998.

In the January 7, 1998 letter, EOI requested relief from ASME Code weld examinations for three specific areas:

- (1) (RR2-0001) ASME Code, Section XI, Examination Category B-A, Item B1-22 requires that essentially 100% of the accessible length of all reactor pressure vessel (RPV) meridional welds be volumetrically examined as defined by Figure IWB-2500-3. However, access to RPV meridional head Welds B13-D001-DG and DH is restricted by the control rod drive assemblies and the RPV support structure. Relief from the ASME Code requirement was requested since imposition of this requirement would cause a considerable burden on the licensee.
- (2) (RR2-0002) ASME Code, Section XI, Examination Category C-B, Item C2.22 requires 100% volumetric examination of Class 2 pressure-retaining nozzle inside radius sections that are either welded to or integrally cast in vessels that connect to piping runs. However, the physical configuration of Nozzle Inner Radius Sections N3 (E12-EB001X-3.743-A) and N4 (E12-EB0011X-3.744-A) for Residual Heat Removal (RHR) Heat Exchangers A, B, C, and D would limit meaningful volumetric examination results. Surface examination would also require the heat exchanger tube bundle to be removed to access the inside radius sections of the nozzles. Therefore, the Code-required volumetric examination is impractical at RBS.

4
DFOI

000082

9811100084 981103
PDR ADOCK 05000458
G PDR

NRC FILE CENTER COPY

- (3) (RR2-0003) ASME Code, Section XI, Examination Category C-G, Item C6.10 requires, in part, that Class 2 pump casing welds receive a 100% surface examination of welds from either the inside or outside surface of the component. EOI requested relief from Code required surface examination of the pressure-retaining welds on the following Class 2 pumps installed in the RHR, High Pressure Core Spray (HPCS) and Low Pressure Core Spray (LPCS) systems:

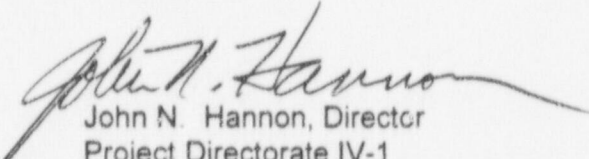
- (a) E12-PC002A (RHR)
- (b) E12-PC002B (RHR)
- (c) E12-PC002C (RHR)
- (d) E22-PC001 (HPCS)
- (e) E21-PC001 (LPCS)

The pump casing welds for the above components are encased in concrete, and would require significant disassembly or redesign of the pump in order to perform the Code-required examinations. Consequently, the imposition of this requirement is impractical and would cause a considerable burden on the licensee.

As required by regulations, the ISI for ASME Code Class 1, 2 and 3 components at nuclear power plants shall be performed in accordance with Section XI of the ASME Code and applicable addenda as required by 10 CFR 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55a(g)(6)(i). Or, as stated in 10 CFR 50.55a(a)(3), alternatives to the requirements of paragraph (g) may be used, when authorized by the Commission, if (i) the proposed alternatives would provide an acceptable level of quality and safety or (ii) compliance with the specified requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.

In the enclosed Safety Evaluation, the staff concludes that your proposed relief from the inservice inspection requirements of the ASME Code is acceptable and is authorized pursuant to 10 CFR 50.55a(g)(6)(i). This is based upon the determination by the Commission, as stated in the Safety Evaluation, that the implementation of proposed alternatives will provide an acceptable level of quality and safety at RBS.

Sincerely,



John N. Hannon, Director
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-458

Enclosure: Safety Evaluation

cc w/encl: See next page

- (3) (RR2-0003) ASME Code, Section XI, Examination Category C-G, Item C6.10 requires, in part, that Class 2 pump casing welds receive a 100% surface examination of welds from either the inside or outside surface of the component. EOI requested relief from Code required surface examination of the pressure-retaining welds on the following Class 2 pumps installed in the RHR, High Pressure Core Spray (HPCS) and Low Pressure Core Spray (LPCS) systems:

- (a) E12-PC002A (RHR)
- (b) E12-PC002B (RHR)
- (c) E12-PC002C (RHR)
- (d) E22-PC001 (HPCS)
- (e) E21-PC001 (LPCS)

The pump casing welds for the above components are encased in concrete, and would require significant disassembly or redesign of the pump in order to perform the Code-required examinations. Consequently, the imposition of this requirement is impractical and would cause a considerable burden on the licensee.

As required by regulations, the ISI for ASME Code Class 1, 2 and 3 components at nuclear power plants shall be performed in accordance with Section XI of the ASME Code and applicable addenda as required by 10 CFR 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55A(g)(6)(i). Or, as stated in 10 CFR 50.55a(a)(3), alternatives to the requirements of paragraph (g) may be used, when authorized by the Commission, if (i) the proposed alternatives would provide as acceptable level of quality and safety or (ii) compliance with the specified requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.

In the enclosed Safety Evaluation, the staff concludes that your proposed relief from the inservice inspection requirements of the ASME Code is acceptable and is authorized pursuant to 10 CFR 50.55a(g)(6)(i). This is based upon the determination by the Commission, as stated in the Safety Evaluation, that the implementation of proposed alternatives will provide an acceptable level of quality and safety at RBS.

Sincerely,
ORIGINAL SIGNED BY:
 John N. Hannon, Director
 Project Directorate IV-1
 Division of Reactor Projects III/IV
 Office of Nuclear Reactor Regulation

Docket No. 50-458

Enclosure: Safety Evaluation

cc w/encl: See next page

DISTRIBUTION: Docket File PUBLIC PD4-1 r/f JHannon
 CHawes RFretz TGwynn, RIV GHill(2) ACRS
 OGC (15B18) THarris(TLH3) EAdensam (EGA1) DLange, RIV

Document Name: RBA0620.REL

OFC	PM/PD4-1	LA/PD4-1	PD/PD4-1
NAM: E	RFretz <i>RF</i>	CHawes <i>CH</i>	JHannon
DATE	11/3/98	11/3/98	11/3/98
COPY	<u>YES</u> /NO	YES/NO	<u>YES</u> /NO

Mr. Randall K. Edington
Entergy Operations, Inc.

River Bend Station

cc:

Winston & Strawn
1400 L Street, N.W.
Washington, DC 20005-3502

Manager - Licensing
Entergy Operations, Inc.
River Bend Station
P. O. Box 220
St. Francisville, LA 70775

Senior Resident Inspector
P. O. Box 1050
St. Francisville, LA 70775

President of West Feliciana
Police Jury
P. O. Box 1921
St. Francisville, LA 70775

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

Ms. H. Anne Plettinger
3456 Villa Rose Drive
Baton Rouge, LA 70806

Administrator
Louisiana Radiation Protection Division
P. O. Box 82135
Baton Rouge, LA 70884-2135

Executive Vice President and
Chief Operating Officer
Entergy Operations, Inc.
P. O. Box 31995
Jackson, MS 39286

General Manager - Plant Operations
Entergy Operations, Inc.
River Bend Station
P. O. Box 220
St. Francisville, LA 70775

Director - Nuclear Safety
Entergy Operations, Inc.
River Bend Station
P. O. Box 220
St. Francisville, LA 70775

Vice President - Operations Support
Entergy Operations, Inc.
P. O. Box 31995
Jackson, MS 39286-1995

Attorney General
State of Louisiana
P. O. Box 94095
Baton Rouge, LA 70804-9095

Wise, Carter, Child & Caraway
P. O. Box 651
Jackson, MS 39205