

November 5, 1998

Mr. James W. Langenbach, Vice President
and Director, TMI
GPU Nuclear, Inc.
P.O. Box 480
Middletown, PA 17057

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING GPU NUCLEAR'S
RESPONSE TO GENERIC LETTER (GL) 96-05 FOR THE THREE MILE ISLAND
NUCLEAR STATION, UNIT NO. 1 (TMI-1)(TAC NO. M97111)

Dear Mr. Langenbach:

The staff has reviewed your letter dated May 13, 1998, regarding GL 96-05, "Periodic Verification of Design-Basis Capability of Safety-Related Motor-Operated Valves." The staff has determined that it will need the additional information identified in the enclosure to complete its review of your response.

During a telephone conference with Mr. Gregory M. Gurican of your staff, it was agreed that a 90 day response to this request is reasonable and within the ability of your staff resources.

Sincerely,

Original signed by

Timothy G. Colburn, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure: Request for Additional Information

cc w/encl: See next page

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J. Langenbach
Three Mile Island Nuclear Station, Unit No. 1

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REQUEST FOR ADDITIONAL INFORMATION
THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1 (TMI-1)
RESPONSE TO GENERIC LETTER (GL) 96-05

1. In NRC Inspection Report No. 50-289/97-07, the NRC staff closed its review of the motor-operated valve (MOV) program implemented at the Three Mile Island Nuclear Station, Unit 1 (TMI-1) in response to Generic Letter (GL) 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance," based on the results of the inspection and the licensee's plans to resolve several outstanding MOV issues as described in a letter dated June 17, 1997. In a letter dated December 31, 1997, the licensee notified the NRC staff that it had completed the commitments specified in its letter dated June 17, 1997, with the exception of some recommendations from its MOV Independent Review Team that were targeted for completion in 1998. The licensee should provide the status of those remaining actions.
2. In a letter dated May 13, 1998, the licensee updated its commitment to implement the Joint Owners Group (JOG) Program on MOV Periodic Verification in response to GL 96-05. The JOG program specifies that the methodology and discrimination criteria for ranking MOVs according to their safety significance are the responsibility of each participating licensee. In a previous letter dated March 18, 1997, the licensee stated that the ranking of MOVs at TMI-1 would be based on the Boiling Water Reactor Owners' Group (BWROG) methodology as described in BWROG Topical Report NEDC 32264 (Revision 2, dated September 1996). As TMI-1 is a pressurized water reactor (PWR) designed by Babcock & Wilcox (B&W), the licensee should describe the application of the BWROG methodology to TMI-1, including (1) the preparation of a sample list of high-risk MOVs from other B&W nuclear plants, and (2) consideration of the conditions and limitations discussed in the NRC safety evaluation dated February 27, 1996, on the BWROG methodology. In responding to this request, the licensee might apply insights from the guidance provided in the Westinghouse Owners Group (WOG) Engineering Report V-EC-1658-A (Revision 2, dated August 13, 1993), "Risk Ranking Approach for Motor-Operated Valves in Response to Generic Letter 96-05," and the NRC safety evaluation dated April 14, 1998, on the WOG methodology for risk ranking MOVs at Westinghouse-designed PWR nuclear plants.
3. The JOG program focuses on the potential age-related increase in the thrust or torque required to operate valves under their design-basis conditions. In the NRC safety evaluation dated October 30, 1997, on the JOG program, the NRC staff specified that licensees are responsible for addressing the thrust or torque delivered by the MOV motor actuator and its potential degradation. The licensee should describe the plan at TMI-1 for ensuring adequate MOV motor actuator output capability, including consideration of recent guidance in Limitorque Technical Update 98-01 and its Supplement 1.

Enclosure