



John C. Brons
Senior Vice President
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April 30, 1986
JPN-86-22

Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Attention: Mr. Daniel R. Muller, Director
BWR Project Directorate No. 2
Division of BWR Licensing

Subject: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Reactor Pressure Vessel Surveillance Materials Program
Summary Report and Implementation Schedule

- References:
1. Code of Federal Regulations, Title 10, Part 50, Appendix H "Reactor Vessel Material Surveillance Program Requirements", 48FR24008, May 31, 1983.
 2. Code of Federal Regulations, Title 10, Part 50, Appendix G "Fracture Toughness Requirements" 48FR24008, May 31, 1983.

Dear Mr. Muller:

Reference 1 requires that the Authority have a program to monitor the fracture toughness properties of ferritic materials in the beltline region of the reactor vessel. Under the program, data are obtained from tests of exposed material samples which are withdrawn periodically from the reactor vessel. Reference 1 also requires that the report of the result of these tests be submitted to the NRC within one year of withdrawal of a reactor vessel sample. A report on the FitzPatrick Reactor Vessel Surveillance Materials Testing and Fracture Toughness Analysis is attached.

Reference 2 requires that reactor vessel beltline materials maintain an End-of-Life (EOL) Charpy upper-shelf energy of no less than 50 ft-lb. The predicted adjusted reference temperature must not exceed 200°F.

A reactor vessel material sample was removed during the Reload 6/Cycle 7 outage. The sample was analyzed, and the EOL adjusted reference temperature was determined to be 104°F. The EOL values of upper shelf energy for plate and weld material were determined to be 58 lb-ft transverse and 72 ft-lb, respectively. Therefore, the requirements of Reference 2 are satisfied. Operating limit curves

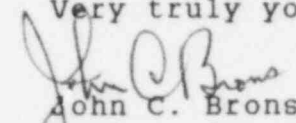
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have been established for three reactor conditions: hydrostatic testing, non-nuclear heatup and cooldown, and core critical operation. The curves are valid for up to 16 effective full power years of operation. The FitzPatrick Technical Specifications will be revised to reflect the new pressure-temperature conditions. The Authority expects to submit revised Technical Specifications in November 1986. Current Technical Specifications are still within requirements and can be used until that time.

Should you or your staff have any further questions regarding this matter, please contact Mr. J. A. Gray, Jr. of my staff.

Very truly yours,



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cc: Office of the Resident Inspector
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