

BEFORE THE
UNITED STATES NUCLEAR REGULATORY COMMISSION

In the Matter of the Facility Operating License)
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)
PUBLIC SERVICE COMPANY OF COLORADO)

Application for Amendment to
the Facility Operating License
License No. DPR-34

OF THE
PUBLIC SERVICE COMPANY OF COLORADO
FOR THE
FORT ST. VRAIN NUCLEAR GENERATING STATION

This application for Amendment to the
Facility Operating License, License No. DPR-34,
is submitted for NRC review and approval.

Respectfully submitted,

PUBLIC SERVICE COMPANY OF COLORADO

By R. O. Williams, Jr.
Vice President
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Attorneys for Applicant

ATTACHMENT 1

SUMMARY OF
PROPOSED CHANGES

Summary of Proposed Changes

<u>Section</u>	<u>Description</u>
License Information	<ol style="list-style-type: none">1) Reformat entire License Information Section, including deletion of the cover letter, the Notice of Issuance and the B-50 Agreement.2) Make the portion of the License Information which covers radioactive material possession similar to more recently licensed plants.

ATTACHMENT 2
PROPOSED CHANGES

PUBLIC SERVICE COMPANY OF COLORADO
FORT ST. VRAIN NUCLEAR GENERATING STATION
DOCKET NO. 50-267
FACILITY OPERATING LICENSE

License No. DPR-34

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for license filed by the Public Service Company of Colorado (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1 and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Fort St. Vrain Nuclear Generating Station (the facility) has been substantially completed in conformity with Provisional Construction Permit No. CPPR-54 and the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - E. The licensee is technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;
 - F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Identity Agreements," of the Commission's regulations;
 - G. The issuance of this operating license will not be inimical to the common defense and security or to the health and safety of the public;

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Facility Operating License No. DPR-34 (subject to the conditions for protection of the environment set forth herein) is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
 - I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, 70, and 73.
2. Facility Operating License No. DPR-34 is hereby issued to the Public Service Company of Colorado to read as follows:
- A. This license applies to the Fort St. Vrain Nuclear Generating Station, a high temperature gas-cooled nuclear reactor and associated equipment (the facility) owned by the Public Service Company of Colorado. The facility is located near Platteville, in Weld County, Colorado, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 15 through 29) and the Environmental Report as supplemented and amended (Supplements 1 through 3).
 - B. This license is subject, for the initial rise to power, to the conditions set forth in Specification LCO 4.9-1 of the Technical Specifications attached hereto as Appendix A.
 - C. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses the Public Service Company of Colorado:
 - (1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location near Platteville in Weld County, Colorado, in accordance with the procedures and limitations set forth in this license;
 - (2) Pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess and use at any time source and special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
 - (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- | (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to
| receive, possess and use in amounts as required any
| byproduct, source or special nuclear material without
| restriction to chemical or physical form, for sample
| analysis or instrument calibration or associated with
| radioactive apparatus or components; and

 - | (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to
| possess, but not separate, such byproduct and special
| nuclear materials as may be produced by the operation of
| the facility authorized herein.
- D. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and the appropriate sections of Parts 70 and 73; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 842 megawatts thermal.

 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

 - (3) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the following Commission-approved documents, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p):
 - a. "Fort St. Vrain Unit No. 1 Security Plan"
New consolidated issue dated September 13, 1979, as revised through Revision 10, dated May 26, 1982 (submitted June 2, 1982). (This document is withheld from public disclosure pursuant to 10 CFR 73.21.)

- b. Fort St. Vrain Guard Training and Qualification Plan (Appendix B to the Security Plan), dated February 1982 as revised April 6, 1982. This Plan shall be followed in accordance with 10 CFR 73.55(b)(4) 60 days after approval by the Commission. All security personnel, as required in the above plans, shall be qualified within two years of this approval. The licensee may make changes to this plan without prior Commission approval if the changes do not decrease the safeguards effectiveness of the plan. The licensee shall maintain records of and submit reports concerning such changes in the same manner as required for changes made to the Safeguards Contingency Plan pursuant to 10 CFR 50.54(p). (This document is withheld from public disclosure pursuant to 10 CFR 73.21.)
 - c. Fort St. Vrain Safeguards Contingency Plan (Appendix C to the Security Plan) dated April 6, 1982, submitted pursuant to 10 CFR 73.40. The Contingency Plan shall be fully implemented, in accordance with 10 CFR 73.40(b). (This document is withheld from public disclosure pursuant to 10 CFR 73.21.)
 - d. Chapter 9, Fort St. Vrain Unit 1 Security Plan dated June 10, 1982 (submitted June 11, 1982 revised to ensure compliance with 10 CFR 73.45). (This chapter is classified CONFIDENTIAL, National Security Information.)
- E. This license is subject to all Federal, State, and local standards imposed pursuant to the requirements of the Federal Water Pollution Control Act of 1972.
3. This license is effective as of the date of issuance and shall expire at midnight, September 17, 2008.

FOR THE ATOMIC ENERGY COMMISSION

"ORIGINAL SIGNED BY:"

A. Giambusso, Deputy Director for
Reactor Projects
Directorate of Licensing

Attachments:

Appendices A and B - Technical Specifications

Date of Issuance: December 21, 1973

ATTACHMENT 3

SIGNIFICANT HAZARDS
CONSIDERATION

SIGNIFICANT HAZARDS CONSIDERATION

Description of Amendment Request:

This amendment request is prompted by the need to have a 500 curie source of Cesium 137 on site to perform calibration of high range detection instrumentation. Amendment 41 to the Fort St. Vrain Technical Specifications directed Public Service Company of Colorado to, at a future time, replace the listing of specific isotopes with a statement similar to that now requested in 2.c.(4). The current Radiological Control Program maintains adequate control of the use and storage of calibration sources.

PSC is also proposing to delete the cover letter, the Notice of Issuance, and the B-50 Agreement. These deleted pages contain no license requirements which are not maintained in the License itself. This will serve to place the Fort St. Vrain License in a format more similar to the recently issued Licenses.

Basis for No Significant Hazards Determination:

The proposed change does not involve a significant hazards consideration because operation of FSV in accordance with this change would not:

- (1) involve a significant increase in the probability or consequences of an accident previously evaluated. The use of sources for calibration purposes does not effect the design or function of any plant system/component. The updating of this license condition to not specify individual isotopes will allow more flexibility, and is being done based on an NRC recommendation.
- (2) create the possibility of a new or different kind of accident from any accident previously evaluated. The calibration of detection instrumentation does not create the possibility of any accident different from those already analyzed. Non-specific designation of the calibration sources will not create any new failure modes.
- (3) involve a significant reduction in a margin of safety. There is no margin of safety associated with calibration source strength.

Further, reformatting the license does not alter the requirements expressed in the License.