



**Northeast  
Nuclear Energy**

Rope Ferry Rd. (Route 156), Waterford, CT 06385

Millstone Nuclear Power Station  
Northeast Nuclear Energy Company  
P.O. Box 128  
Waterford, CT 06385-0128  
(860) 447-1791  
Fax (860) 444-4277

The Northeast Utilities System

OCT 30 1998

Docket No. 50-423  
B17489

Re: 10CFR50.90  
10CFR50.59 (a)(2)

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 3  
Proposed License Amendment Request  
Post Accident Access to Vital Areas (PLAR 3-98-6)  
Request for Additional Information

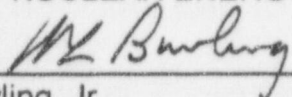
Northeast Nuclear Energy Company (NNECO), in a letter dated June 10, 1998, requested approval of a proposed amendment to Chapter 12 of Millstone Unit No. 3 Final Safety Analysis Report. The NRC, in a letter dated October 20, 1998, requested additional information to support their review of the submittal. Attachment 1 contains NNECO's response.

There are no commitments contained in this letter.

If the NRC Staff should have any questions or comments regarding this submittal, please contact Mr. D. Smith at (860) 437-5840. 1/1

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY A/C 1

  
\_\_\_\_\_  
Martin L. Bowling, Jr.  
Recovery Officer - Technical Services

9811060157 981030  
PDR ADOCK 05000423  
P PDR

cc: H. J. Miller, Region I Administrator  
J. W. Andersen, NRC Project Manager, Millstone Unit No. 3  
A. C. Cerne, Senior Resident Inspector, Millstone Unit No. 3  
E. V. Imbro, Director, Millstone ICAVP Inspections

Docket No. 50-423  
B17489

Attachment 1

Millstone Nuclear Power Station, Unit No. 3  
Request for Additional Information

October 1998

Millstone Nuclear Power Station, Unit No. 3  
Request for Additional Information

**Question 1:**

Provide the integrated dose for each activity, including the dose associated with the primary and alternate egress routes adjusted for decay.

**Response:**

The integrated dose for each activity, including the dose associated with the primary and alternate egress routes adjusted for decay are contained in Table 1 and Table 2.

**Question 2:**

Confirm that the source term used in the calculations is consistent with NUREG 0737, Item II.B.2.

**Response:**

The calculations associated with the doses contained in Table 1 and 2 used a pressurized liquid source term of 100% noble gases, 50% iodines and 1% solid fission products, a depressurized liquid source term of 50% iodines and 1% solid fission products and a gas source term of 100% noble gases and 50% iodines. These source terms are consistent or conservative relative to the guidance contained in NUREG 0737, Item II.B.2.

Table 1  
 DOSE CALCULATION SUMMARY

ACCIDENT MITIGATION TASK		Whole Body Dose (Rem)	
		Primary Route	Alternate Route
1.	Local tripping of the reactor trip breakers and bypass breakers in the 43' 6" level of the Auxiliary Building in the MCC Rod Control Area	2.91	2.61
2.	PASS Sample:	<5	<5
3.	Local realignment of spent fuel pool cooling RBCCW and service water for spent fuel pool cooling in the Spent Fuel Building	0.35	0.66
4.	Powering of the Plant Process Computer from the Turbine Building	1.35	NA
5.	Powering of the safety injection accumulator valves in the 24' level of the Auxiliary Building	4.62	4.46
6.	Initiation of the hydrogen monitor in the Hydrogen Recombiner Building	3.25	4.02
7.	Initiation of the hydrogen monitor in the Hydrogen Recombiner Building	2.08	2.63
8.	Initiation of the hydrogen monitor from the 4', 24'6" and 43'6" levels of the Auxiliary Building	17.7	17.7
9.	Local opening of the breakers for RWST/charging pump suction valves in the 24'6" level of the Auxiliary Building	4.62	4.46
10.	Installation and operation of air compressors for RSS sump pumps outside of the ESF Building east wall	3.6	3.6
11.	Opening of the breakers for non-safety grade sump pumps <ul style="list-style-type: none"> <li>• from within the 21' level of the ESF Building</li> <li>• from within the 24'6" level of the Auxiliary Building</li> </ul>	0.23 0.38	0.23 0.39
12.	Tripping of non-QA fans that may still be operating and venting of SLCRS damper leakage from the 43'6" level of the Auxiliary Building	1.81	1.65
13.	Alignment of Service Water to Auxiliary Feedwater to provide long-term decay heat removal <ul style="list-style-type: none"> <li>• ESF Motor-Driven Pump Compartment</li> <li>• ESF Terry Turbine Compartment</li> </ul>	4.1 4.0	4.1 4.0
14.	Closing of SLCRS Doors ( <b>Not a required Action</b> )	NA	NA
15.	Resetting of MCC Breakers for Diesel Generator Keep Warm Systems	0.52	NA

Table 2  
PASS Sample Collection Doses

Task & Condition		Dose (rem)
1.	Primary Route - 4 timed Collection of PASS samples with the Hydrogen Recombiner Operating w/o Laboratory Analysis	4.30
2.	Alternate Route - 4 timed Collection of PASS Samples - HRB, w/o Laboratory Analysis	4.61
3.	Alternate Route - 4a timed Collection of PASS Samples - HRB, w/o Laboratory Analysis	4.76
4.	Alternate Route - 4b timed Collection of PASS Samples - HRB, w/o Laboratory Analysis, Calculated	4.67
5.	PASS Sample Collection Hydrogen Recombiner Off-Line w/ Laboratory Analysis	0.63
6.	Laboratory Analysis Only	0.38
7.	PASS Sample Collection Excluding Transit Doses, and including Laboratory Analysis with HRB Operating	3.80