U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Reports No. 50-266/86003(DRP); 50-301/86003(DRP)

Docket Nos. 50-266; 50-301 Licenses No. DPR-24; DPR-27

Licensee: Wisconsin Electric Company
231 West Michigan
Milwaukee, WI 53203

Facility Name: Point Beach Unit 1 and 2

Inspection At: Two Creeks, WI

Inspection Conducted: February 1 through March 31, 1986

Inspectors: R. L. Hague
R. J. Leemon

Approved By: I. N. Jackiw, Chief
Reactor Projects Section 2B

4-14-86
Date

Inspection Summary

Inspection on February 1 through March 31, 1986 (Reports No. 50-266/86003(DRP); 50-301/86003(DRP))
Areas Inspected: Routine, unannounced inspection by resident inspectors of licensee action on previous inspection findings; operational safety; surveillance; maintenance; offsite review committee; preparation for refueling; IE bulletin follow-up; and licensee event report follow-up. The inspection involved a total of 288 inspector-hours onsite by two inspectors including 52 inspector-hours on off-shifts.
Results: No violations or deviations were identified.

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DETAILS

1. Persons Contacted

*J. J. Zach, Manager, PBNP
T. J. Koehler, General Superintendent
G. J. Maxfield, Superintendent, Operation
*J. C. Reisenbuechler, Superintendent, EQR
W. J. Herrman, Superintendent, Maintenance and Construction
R. S. Bredvad, Health Physicist
R. Krukowski, Security Supervisor
*F. A. Flentje, Staff Services Supervisor
*J. E. Knorr, Regulatory Engineer

The inspector also talked with and interviewed members of the Operation, Maintenance, Health Physics, and Instrument and Control Sections.

*Denotes personnel attending exit interviews.

2. Licensee Action on Previous Inspection Findings (92701)

(CLOSED) Unresolved Item (266/85022-01; 301/85021-01): The region has determined that there was no violation of reporting requirements during the December 31, 1985, Unusual Event. The licensee committed to training required individuals on the use of the new reporting form. This training has been completed.

Regional Request - The inspectors evaluated the licensee's response to the NRC's letter of November 26, 1985, concerning acceptability of equipment procured from Exo Sensors, Incorporated. There were no problems found which required resolution.

3. Operational Safety Verification and Engineered Safety Features System Walkdown (71709 and 71710)

The inspectors observed control room operations, reviewed applicable logs and conducted discussions with control room operators during the months of February and March. During these discussions and observations, the inspectors ascertained that the operators were alert, cognizant of plant conditions, attentive to changes in those conditions, and took prompt action when appropriate. The inspectors verified the operability of selected emergency systems, reviewed tagout records and verified proper return to service of affected components. Tours of the Auxiliary Building, Turbine Building and Pumphouse were conducted to observe plant equipment conditions, including potential fire hazards, fluid leaks, and excessive vibrations and to verify that maintenance requests had been initiated for equipment in need of maintenance.

The inspectors, by observation and direct interview, verified that the physical security plant was being implemented in accordance with the station security plan and noted one potential violation. This item will be covered in a special security inspection Reports No. 266/86005, No. 301/86005.

The inspectors observed plant housekeeping/cleanliness conditions and verified implementation of radiation protection controls. During the months of February and March, the inspectors walked down the accessible portions of the Auxiliary Feedwater, Vital Electrical, Diesel Generator, Component Cooling, Safety Injection, Containment Spray, Service Water and Fire systems to verify operability.

These reviews and observations were conducted to verify that facility operations were in conformance with the requirements established under Technical Specifications, 10 CFR and administrative procedures.

During a walkdown of the control boards at 4:00 p.m. on March 6, 1986, the control room operator discovered the indicating lights for two low head safety injection (SI) valves on train A were not lit. Investigation revealed that a protective covering for the 480 volt motor control center fell from scaffolding at about 1:30 p.m. on March 6, 1986, and the contractors failed to notify the control room or their supervisor. At 4:10 p.m. the breakers were reclosed and the breaker cubicle was inspected and no damage was found. Further licensee corrective action included revision of the operators log to include switch position and power available check-offs for all valves that are required to function during an SI and a retraining of the contractors on the importance of notifying the control room of any other than normal occurrences.

No violations or deviations were identified.

4. Monthly Surveillance Observation (61726)

The inspector observed technical specifications required surveillance testing on the Reactor Protection and Safeguards Analog Channels and Nuclear Instrumentation and verified that testing was performed in accordance with adequate procedures, that test instrumentation was calibrated, that limiting conditions for operation were met, that removal and restoration of the affected components were accomplished, that test results conformed with technical specifications and procedure requirements and were reviewed by personnel other than the individual directing the test, and that any deficiencies identified during the testing were properly reviewed and resolved by appropriate management personnel.

The inspector also witnessed or reviewed portions of the following test activities:

IT-03 "Inservice Testing of Low Head Safety Injection Pumps and Valves, Unit 1"

IT-04 "Inservice Testing of Low Head Safety Injection Pumps and Valves, Unit 2"

IT-07 "Inservice Testing of Service Water Pump and Valves, Units 1 and 2"

IT-08	"Inservice Testing of Turbine-Driven Auxiliary Feedwater Pump"
ICP 2.1	"Periodic Test Reactor Protection and Safeguards Analog Channel I through IV"
WMTP 6.1	"Core Power Distribution and Nuclear Power Detector Calibration Checks"
WMTP 9.2	"Nuclear Power Range Detector Calibration Quarterly Axial Offset Test"

No violations or deviations were identified.

5. Monthly Maintenance Observation (62703)

Station maintenance activities on safety related systems and components listed below were observed/reviewed to ascertain that they were conducted in accordance with approved procedures, regulatory guides and industry codes or standards and in conformance with technical specifications.

The following items were considered during this review: the limiting conditions for operation were met while components or systems were removed from service; approvals were obtained prior to initiating the work; activities were accomplished using approved procedures and were inspected as applicable; functional testing and/or calibrations were performed prior to returning components or systems to service; quality control records were maintained; activities were accomplished by qualified personnel; parts and materials used were properly certified; radiological controls were implemented; and fire prevention controls were implemented.

Work requests were reviewed to determine status of outstanding jobs and to assure that priority is assigned to safety related equipment maintenance which may affect system performance.

The following maintenance activities were observed/reviewed:

Repair of boric acid heat tracing circuit (P-131) on the manual borate line.

The Diesel Generator (3D) annual inspection.

Repair of service water valve (SW-2839) for diesel generator (3D) oil cooler.

Repair of blowdown sample valve (CV-2093).

Replacement of internal wiring in Limitorque operators with environmentally qualified wire (5 of 40 operators have been rewired as of March 31, 1986).

Changing of oil in Unit 2 turbine-driven auxiliary feedwater pump.

Electric fire pump piping restraint.

Changing of a single cell in "A" battery.

Adjusting of manual trip linkage on Unit 1 turbine-driven auxiliary feedwater pump.

As requested by Region III, both diesel generators were inspected for cracks on the diesel accessory drive housing assembly, which supports various shaft-driven pumps, to detect cracks on the drive housing between the bolt holes and a steel insert that is press fit into the aluminum assembly. The accessory drive housing was checked for cracks around the pressed in steel inserts, and no cracks were found. The turbocharger end was also checked for cracks, none were found.

The last time the accessory drive housing was removed from 4D was in 1980 to replace the vibration dampener (Maintenance Work Request No. 24246).

The last time 3D's housing was removed was in 1981 to replace the vibration dampener (Maintenance Work Request No. 30022). No cracking noted on the Maintenance Work Request.

No violations or deviations were identified.

6. Offsite Review Committee (40701)

The inspector reviewed the licensee's commitments with respect to the composition, duties and responsibilities of and the administrative controls over the Offsite Review Committee.

The inspector attended a portion of the Offsite Review Committee meeting and reviewed the minutes of this meeting.

The inspector evaluated the proper performance of the committee reviews of: licensee event reports; violations of regulations or deviations from licensee commitment, licensee procedures, and orders; proposals that could affect nuclear safety; procedural, equipment, or system changes; changes to Technical Specifications (TS) or to the Operating License (OL), and tests or experiments; significant operating abnormalities or component design deficiencies; and the onsite review committee minutes and reports.

The inspector determined that the committee's composition with respect to disciplines and expertise required by the Technical Specifications was satisfied and that a quorum was present during the meeting.

No violations or deviations were identified.

7. Preparation for Refueling (60705)

The inspector verified that technically adequate procedures were approved for Unit 1, Refueling 13. Unit 1 is scheduled to go off line on April 11, 1986. The inspector verified that the licensee had submitted a proposed core reload technical specification change to NRR (or that the licensee's 10 CFR 50.59 safety evaluation of the reload core showed that prior NRR review is not required). The inspector also reviewed the licensee's program for overall outage control.

The inspector verified, prior to receipt of new fuel, that technically adequate, approved procedures were available covering the receipt, inspection, and storage of new fuel; observed receipt inspections and storage of new fuel elements and verified these activities were performed in accordance with the licensee's procedures: RP-2A, Receipt of New Fuel Assemblies; RP-2B, Damaged New Fuel Assembly and Insert Visual Inspection; RP-2C, New Fuel and Fuel Insert Inspection; RP-2D, New RCCA Receipt.

No violations or deviations were identified.

8. IE Bulletin Followup (92701)

For the IE Bulletin listed below the inspector verified that the written response was within the time period stated in the bulletin, that the written response included the information required to be reported, that the written response included adequate corrective action commitments based on information present in the bulletin and the licensee's response, that licensee management forwarded copies of the written response to the appropriate onsite management representatives, that information discussed in the licensee's written response was accurate, and that corrective action taken by the licensee was as described in the written response.

85-01 Steam Binding of Auxiliary Feedwater Pumps

The inspectors verified, as required, all of the inspection requirements of T.I. 2515/69.

No violations or deviations were identified.

9. Licensee Event Reports Followup (92700)

Through direct observations, discussions with licensee personnel, and review of records, the following event report was reviewed to determine that reportability requirements were fulfilled, immediate corrective action was accomplished, and corrective action to prevent recurrence had been accomplished in accordance with technical specifications.

Unit 2, Docket No. 50-301
Licensee Event Report No. 85-005-00 - Reactor Trip Due to Loss of Load.

No violations or deviations were identified.

10. Exit Interview (30703)

The inspectors met with licensee representatives (denoted in Paragraph 1) throughout the inspection period and at the conclusion of the inspection period to summarize the scope and findings of the inspection activities. The licensee acknowledged the inspectors' comments. The inspectors also discussed the likely informational content of the inspection report with regard to documents or processes reviewed by the inspectors during the inspection. The licensee did not identify any such documents/processes as proprietary.