

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
MAY 01 1975

Docket Nos. 50-416
and 50-417

Applicant: Mississippi Power & Light Company

Facility : Grand Gulf Nuclear Station Units 1 and 2

SUMMARY OF MEETING HELD ON APRIL 23, 1975 TO DISCUSS MARK III CONTAINMENT
DESIGN BASES

A brief unscheduled meeting was held on April 23, 1975, at the NRC offices in Bethesda, Maryland. The meeting was requested by the applicants to discuss their proposed response to the NRC letter of March 3, 1975, requesting information on the design basis loading for containment structures as they are modified from the design presented in the PSAR. An attendance list is provided in the Enclosure.

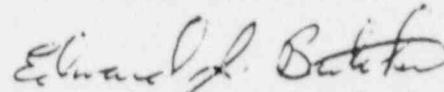
The applicants stated that the design of the safety/relief valve discharge system has been changed to the standard G.E. design with the rams head discharge, and that the containment design loading will be that provided by G.E. The NRC staff asked how the G.E. furnished loads would be justified. The applicants replied, by reference to the G.E. Topical Report NEDO-10859 with certain refinements. The NRC staff stated its position that a reference to NEDO-10859 alone would not be adequate justification for the loads and that written justification would be required either on the Grand Gulf docket or by a suitable reference. A three part approach to justifying the loads was discussed as follows:

- 1) Written documentation of the refinements to the calculational methods presented in NEDO-10859 should be provided, taking into consideration all of the NRC staff concerns outlined in its evaluation of NEDO-10859 dated September 12, 1974;
- 2) The results of recent tests performed to determine safety relief valve discharge loads should be provided; and
- 3) A commitment to perform follow-up in plant tests to verify calculated loads should be made.



Suppression pool swell loads were also discussed. The applicants stated that the additional test data requested by the NRC staff would have to come from the G.E. test facility and would not be available for NRC review until September 1975 due to G.E. manpower limitations. They also stated that they intend to begin placing concrete for the containment base mat on June 26, 1975 and that structures up to the 135 ft elevation will be in place by January 1976. The NRC staff noted that structures as low as the 119 ft elevation level are subject to suppression pool swell loads and that the test data necessary to fully justify the design basis loading at this elevation has not been provided to the NRC by G.E. The applicants intend to proceed with construction without consideration of the additional test data, that won't be available until September 1975, because they feel adequate conservatism is built into the containment structures design and the structures are capable of being strengthened at a later date, if found to be required.

The NRC staff summarized the status of its review of the Grand Gulf containment design by stating that the information provided to date by G.E. and the applicants was not adequate to justify the loads that have been specified. In order for the NRC staff to complete its evaluation of this matter, a complete response to its letters, of March 3, 1975 on pool dynamic loads and April 23, 1975 on safety/relief valve discharge loads, will be required.



Edward J. Butcher, Project Engineer
Light Water Reactors Branch 1-2
Division of Reactor Licensing

Enclosure:
Attendance List

ATTENDANCE LIST

<u>NAME</u>	<u>ORGANIZATION</u>
E. J. Butcher	NRC
C. L. Cudlin	NRC
J. A. Kudrick	NRC
G. C. Lainas	NRC
L. Slegers	NRC
J. P. McGaughv	MP&L
C. L. Reid	Bechtel
R. P. Barr	G.E.