



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 6, 1988

Docket No. 50-302

Mr. W. S. Wilgus  
Vice President, Nuclear Operations  
Florida Power Corporation  
ATTN: Manager, Nuclear Licensing  
P. O. Box 219  
Crystal River, Florida 32629

Dear Mr. Wilgus:

SUBJECT: CRYSTAL RIVER UNIT 3 - RELIEF FROM INSERVICE TESTING  
REQUIREMENTS (TAC NO. 69002)

By letter dated July 13, 1988, Florida Power Corporation requested relief from certain pump and valve inservice testing requirements of the ASME Boiler and Pressure Vessel Code, Section XI, 1983 Edition through Summer 1983 Addenda (Code), as follows:

<u>Relief Request Number</u>	<u>Description of Relief Requested</u>
V-113	Relief from Code Section IWP-3100 requirements for measuring bearing temperature and vibration amplitude for all pumps, and substitution of alternate methods of monitoring for pump degradation.
V-191	Relief from Code Section IWV-3522 to full stroke certain valves during each cold shutdown, and substitution of alternate examination schedules.
V-192	Relief from Code Section IWV-3522 to full stroke certain valves during each cold shutdown, and substitution of alternate examination schedules.

We have reviewed these requests and, based on our review as summarized in the enclosed safety evaluation, herewith grant the requested reliefs.

Please note that the valves covered by Relief Requests V-191 and V-192 must also be partially stroked and individually back-seat tested quarterly.

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For Relief Request V-113, we have determined, in accordance with 10 CFR 55.55a(a)(3)(i), that the alternative proposed by FPC provides an acceptable level of quality and safety. For Relief Requests V-191 and V-102, we have determined, in accordance with 10 CFR 50.55a(g)(6)(i), that the Code requirements are impractical and that the granting of the reliefs is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest, giving due consideration to the burden upon the licensee that could result if the requirements were imposed on the facility.

Sincerely,

*HS*

Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
As stated

cc w/enclosure:  
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*DF01*

Mr. W. S. Wilgus  
Florida Power Corporation

Crystal River Unit No. 3 Nuclear  
Generating Plant

cc:

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